

13 03/09/79

REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)
DISTRIBUTION FOR INCOMING MATERIAL

50-260

REC: OREILLY J P
NRC

ORG: FOX H S
TN VALLEY AUTH

DOCDATE: 02/28/78
DATE RCVD: 03/03/78

DOCTYPE: LETTER NOTARIZED: NO
SUBJECT:

COPIES RECEIVED
LTR 1 ENCL 1

LICENSEE EVENT REPT (RO 50-260/78-004) ON 02/13/78 CONCERNING
DURING STEADY STATE OPERATION, RELIEF VALVE 2-1-5 OPENED AND FAILED
TO RESEAT.

PLANT NAME: BROWNS FERRY - UNIT 2

REVIEWER INITIAL: XJM
DISTRIBUTOR INITIAL:

***** DISTRIBUTION OF THIS MATERIAL IS AS FOLLOWS *****

INCIDENT REPORTS
(DISTRIBUTION CODE A002)

FOR ACTION: BR CHIEF-LEAR**W/4 ENCL

INTERNAL: REG. FILE**W/ENCL
I & E**W/2 ENCL
SCHROEDER/IPPOLITO**W/ENCL
NOVAK/CHECK**W/ENCL
KNIGHT**W/ENCL
HANAUER**W/ENCL
EISENHUT**W/ENCL
SHAO**W/ENCL
KREGER/J. COLLINS**W/ENCL
K SEYFRIT/IE**W/ENCL

NRC PDR**W/ENCL
MIPC**W/3 ENCL
HOUSTON**W/ENCL
GRIMES**W/ENCL
BUTLER**W/ENCL
TEDESCO**W/ENCL
BAER**W/ENCL
VOLLMER/BUNCH**W/ENCL
ROSA**W/ENCL

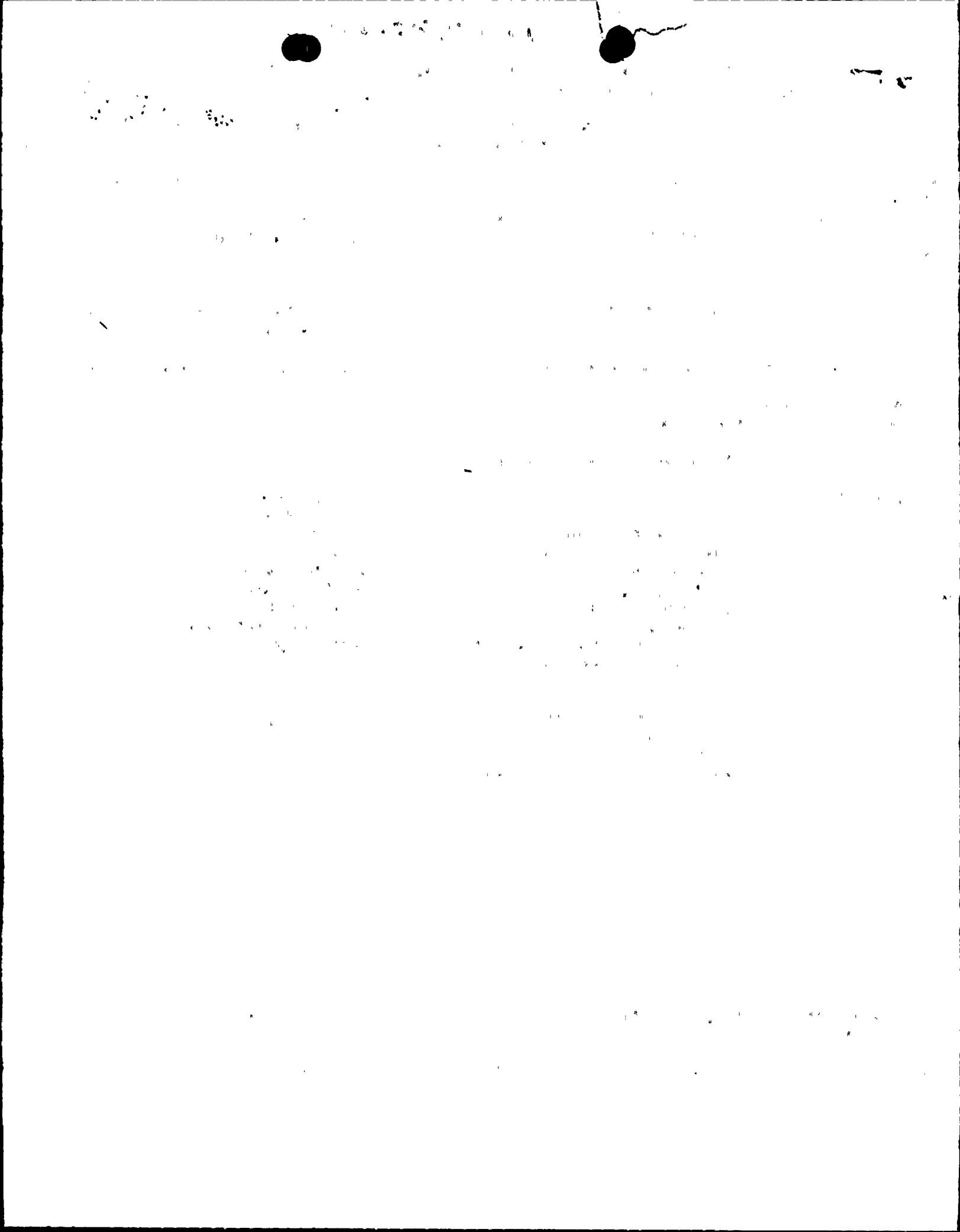
EXTERNAL: LPDR'S
ATHENS, AL**W/ENCL
TIC**W/ENCL
NSIC**W/ENCL
ACRS CAT B**W/16 ENCL

DISTRIBUTION: LTR 45 ENCL 45
SIZE: 1P+1P

CONTROL NBR: 780660013

***** THE END *****

W



TENNESSEE VALLEY AUTHORITY

CHATTANOOGA, TENNESSEE 37402

FEB 28 1978

REGULATORY DOCKET FILE COPY



Mr. James P. O'Reilly, Director
U.S. Nuclear Regulatory Commission
Office of Inspection and Enforcement
Region II
230 Peachtree Street, NW., Suite 1217
Atlanta, Georgia 30303

Dear Mr. O'Reilly:

TENNESSEE VALLEY AUTHORITY - BROWNS FERRY NUCLEAR PLANT UNIT 2 -
DOCKET NO. 50-260 - FACILITY OPERATING LICENSE DPR-52 - REPORTABLE
OCCURRENCE REPORT BFRO-50-260/784

The enclosed report provides details concerning relief valve 2-1-5 which
opened and failed to reseal during steady state operation. This report
is submitted in accordance with Browns Ferry unit 2 Technical Specifications,
Sections 6.7.2.a.(2) and 6.7.2.a.(9).

Very truly yours,

TENNESSEE VALLEY AUTHORITY

H. S. Fox
Director of Power Production

Enclosure
cc (Enclosure):

Director (3)
Office of Management Information and Program Control
U.S. Nuclear Regulatory Commission
Washington, DC 20555

Director (40)
Office of Inspection and Enforcement
U.S. Nuclear Regulatory Commission
Washington, DC 20555

780660013

A002
S
1/1

LICENSEE EVENT REPORT

EXHIBIT A

| | | | | | | | | | | | | | | | | | | | | | |
|--|--|---|--|--|--|--|--|--|--|---|--|--|--|--|--|--|--|--|--|--|--|
| CONTROL BLOCK: 1 | | | | | | | | | | (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION) | | | | | | | | | | | |
| 01 | | A L B R F 2 2 0 0 - 0 0 0 0 0 - 0 0 3 4 1 1 1 1 4 5 | | | | | | | | | | LICENSEE CODE LICENSE NUMBER LICENSE TYPE CAT 58 | | | | | | | | | |
| CONT 01 | | REPORT SOURCE L 6 0 5 0 0 0 2 6 0 7 0 2 1 3 7 8 8 0 2 2 7 7 8 9 | | | | | | | | | | DOCKET NUMBER EVENT DATE REPORT DATE | | | | | | | | | |
| EVENT DESCRIPTION AND PROBABLE CONSEQUENCES 10 | | | | | | | | | | | | | | | | | | | | | |
| 02 | | During steady state operation, relief valve 2-1-5 opened and failed to | | | | | | | | | | | | | | | | | | | |
| 03 | | reseat. The unit was tripped, and the valve reseated at 195 psig. The | | | | | | | | | | | | | | | | | | | |
| 04 | | moderator cooldown rate exceeded the 100° F./hr. rate allowed by Technical | | | | | | | | | | | | | | | | | | | |
| 05 | | Specification 3.6.A.1. Similar failures of valves to reseal have | | | | | | | | | | | | | | | | | | | |
| 06 | | previously been reported in BFAO-50-260/7349, BFAO-50-260/749, | | | | | | | | | | | | | | | | | | | |
| 07 | | BFAO-50-260/7429, BFAO-50-260/7430, and BFAO-50-260/783. There was no | | | | | | | | | | | | | | | | | | | |
| 08 | | hazard to the public health and safety. | | | | | | | | | | | | | | | | | | | |
| 09 | | SYSTEM CODE C 9 11 | | | | | | | | | | CAUSE CODE E 12 CAUSE SUBCODE B 13 COMPONENT CODE V A L V E X 14 | | | | | | | | | |
| | | COMP. SUBCODE X 15 VALVE SUBCODE H 16 | | | | | | | | | | | | | | | | | | | |
| 17 | | LER/RO REPORT NUMBER 7 8 | | | | | | | | | | EVENT YEAR 7 8 | | | | | | | | | |
| | | SEQUENCE REPORT NO. 0 0 4 | | | | | | | | | | OCCURRENCE CODE 0 1 | | | | | | | | | |
| | | REPORT TYPE T | | | | | | | | | | REVISION NO. 0 | | | | | | | | | |
| 18 | | ACTION TAKEN C 18 | | | | | | | | | | FUTURE ACTION Z 19 | | | | | | | | | |
| | | EFFECT ON PLANT A 20 | | | | | | | | | | SHUTDOWN METHOD B 21 | | | | | | | | | |
| | | HOURS 0 0 4 0 | | | | | | | | | | ATTACHMENT SUBMITTED N 23 | | | | | | | | | |
| | | NPRD FORM SUB. N 24 | | | | | | | | | | PRIME COMP. SUPPLIER N 25 | | | | | | | | | |
| | | COMPONENT MANUFACTURER T 0 2 0 | | | | | | | | | | | | | | | | | | | |
| CAUSE DESCRIPTION AND CORRECTIVE ACTIONS 27 | | | | | | | | | | | | | | | | | | | | | |
| 10 | | The valve which malfunctioned was a Target Rock, model 67F, serial No. 62. | | | | | | | | | | | | | | | | | | | |
| 11 | | The cause of the malfunction is not known at this time. Tests and in- | | | | | | | | | | | | | | | | | | | |
| 12 | | spections will be performed to determine the cause and results will be | | | | | | | | | | | | | | | | | | | |
| 13 | | reported. | | | | | | | | | | | | | | | | | | | |
| 14 | | | | | | | | | | | | | | | | | | | | | |
| 15 | | FACILITY STATUS E 28 | | | | | | | | | | % POWER 0 8 6 29 | | | | | | | | | |
| | | OTHER STATUS N/A 30 | | | | | | | | | | METHOD OF DISCOVERY A 31 | | | | | | | | | |

***Revision**

