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September 20, 2018

Mr. Dave Garmon, Health Physicist Radiation Protection and Consequence Branch U.S. Nuclear Regulatory Commission Washington, DC 20555-0001

Subject: Industry Comments on NRC Draft Revision to the Public Radiation Safety Significant Determination Process

Project Number: 689

Dear Mr. Garmon:

On behalf of the Nuclear Energy Institute's (NEI)¹ members, we provide the attached comments on the NRC Draft Revision to the Public Radiation Safety Significant Determination Process provided to external stakeholders.

We thank you for the opportunity to provide the nuclear energy industry's comments on the Draft Revision. If you have any questions or require additional information, please contact me.

Sincerely,

W. How

Jerry W. Hiatt, CHP

Attachment

c: Dr. Kevin Hsueh, NRR/NRC

¹The Nuclear Energy Institute (NEI) is the organization responsible for establishing unified industry policy on matters affecting the nuclear energy industry, including the regulatory aspects of generic operational and technical issues. NEI's members include entities licensed to operate commercial nuclear power plants in the United States, nuclear plant designers, major architect/engineering firms, fuel cycle facilities, nuclear materials licensees, and other organizations and entities involved in the nuclear energy industry.

Summary of Industry Comments 09.20.2018

USNRC PUBLIC RADIATION SAFETY SIGNIFICANCE DETERMINATION PROCESS DRAFT REVISION FOR EXTERNAL STAKEHOLDERS,

Page-Location – NRC Draft	Proposed Text in NRC Draft	Industry Comment
p.3, 1 st full paragraph	If the licensee has a substantial failure to implement the radioactive effluent release program, then the finding would be WHITE	Comment : Regarding the WHITE designation – Is there no consideration of the actual public dose resulting from the failure? Should not a minimal dose consequence be considered GREEN?
p.3, 2 nd paragraph	If the licensee has a substantial failure to implement the radioactive effluent release program, then the finding would be WHITE. <i>Failure to identify a release</i> <i>event</i> , or assess the dose consequences and the impact to the environment in a timely manner, consistent with ODCM requirements, could be considered a substantial failure to implement the radioactive effluent release program.	Recommendation: Insert the word "radiological" as shown below "Failure to identify a radiological release event"
Page 3, 3 rd Paragraph	Examples of a substantial failure to implement the radioactive effluent release program are:	The uses of the words "significant" and "gross inability" are subjective and subject to inspector interpretation.
	• <i>Significant deficiency</i> in implementing the effluent release program as defined in the plant's Technical Specifications, resulting in the <i>gross inability or gross inaccuracy</i> in characterizing an effluent release.	 Significant deficiency in implementing the effluent release program as defined in the plant's Technical Specifications, resulting in the inability or inaccuracy greater than a factor of 100 in characterizing an effluent release vs. an equivalent NRC. Calculation
	• <i>Significant deficiency</i> in evaluating an effluent release (either planned or unplanned) where the resulting dose has been grossly underestimated.	• Significant deficiency in evaluating a radiological effluent release (either planned or unplanned) where the calculated dose has been underestimated by a factor of 100 vs. an equivalent NRC calculation
	• <i>Significant deficiency</i> in calibrating effluent monitors used to assess effluent	 Significant deficiency in calibrating instrumentation or monitors used to

	 releases, resulting in a gross inability or gross inaccuracy in characterizing an effluent release. Failure to have any data by which to assess the dose to a member of the public from an effluent release (i.e., no monitor data, no independent sample data, no actual release sample data 	 assess radiological effluent releases, resulting in an inability or inaccuracy greater than a factor of 100 vs. equivalent NRC calculation in characterizing an effluent release. Failure to collect and assemble data by which to assess the dose to a member of the public from a radiological effluent release (e.g., no monitor data, no independent sample data no actual
	etc.)	release sample data, etc.)
Overall Section – General Industry	RADIOACTIVE EFFLUENT RELEASE PROGRAM	Comment: The Effluent Release Program section is silent on Solid Radwaste Effluent Reporting. It must be assumed that all findings on Solid Radwaste Effluent reporting would be Green.
Comment		Since the last NRC Rad Effluent Report (2010, which is 7 years late) also excluded any Solid Radwaste Effluent information, this reporting must not be of safety significance. Commercial Solid Low Level Waste disposal data is obtained and stored in the National DOE Manifest Information Management System (MIMS).Therefore, NRR should follow up on its written promise dated June 18, 2015 to revise RG 1.21 with regard to solid radioactive waste shipped. That revision <u>should eliminate the</u> <u>requirement</u> to submit Solid Radwaste shipment quantities as part of the Annual Radiological Effluent Release Report (ARERR). In addition, review of the ARERR for types and amounts of radioactive waste disposed should be deleted from Inspection Manual
nn 6 9 7	C CDD Determination Process	71124.08-03 b.
pp. 6 & 7 General Industry Comment	a. Packaging	Comment: Industry suggests that the definition of what constitutes 1 R/hr at 3 meters be added as Section V C. b. Radiation Limits Exceeded. The draft revision to this section already added clarification regarding averaging radiation levels over the area of a probe. Additional text should be added on averaging 3 meter dose rates from packages.

		Such guidance is needed in this document
		stathis time has seen NUDEC 1000 is not
		at this time because NUREG 1608 is not
		slated for revision for at least 2 years.
р. 9,	2 The loss of package contents means	Comment/Question: what constitutes a
footnote	that radioactive material has been	"low background area?" Is it intended to be
	released and can be detected and	near the location of the potential release or
	distinguished in a low background area	could the package be taken to another
	at a distance of 30 cm from the item	location where the "background" could be
	with a micro-rem per hour-type	lower? e.g., inside a separate building
	instrument, which typically uses a 1 inch	
	by 1 inch scintillation detector.	
р. 12,	It is also expected that the inspector	Comment/Question: This statement
2 nd paragraph	follows previous guidance concerning	indicates that only a single finding would be
	multiple findings on a single incident.	issued. In other words only a "Yellow" and
	That is, a finding with a package breach	not a "Yellow" for the breach and a
	which resulted in a YELLOW	"Green" for the documentation associated
	determination and a CoC deficiency	with the CoC. Is this correct?
	which resulted in a GREEN	
	determination would be considered to	
	be a YELLOW finding. This is because the	
	YELLOW signifies a more serious	
	problem with the package breach aspect	
	of the finding, than the CoC deficiency	
	aspect of the finding.	

Garmon-Candelaria, David

From:	HIATT, Jerry <jwh@nei.org></jwh@nei.org>	
Sent:	Thursday, September 20, 2018 1:13 PM	
То:	Garmon-Candelaria, David	
Cc:	Hsueh, Kevin	
Subject:	[External_Sender] Industry Comments on NRC Draft Revision to the Public Radiation	
	Safety Significant Determination Process	
Attachments:	09-20-18_NRC_NEI Comments on Draft Public Exposure SDP_attachment.pdf; 09-20-18 _NRC_NEI Comments on Draft Public Exposure SDP.PDF	

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Sincerely,

Jerry Hiatt Senior Project Manager, Radiological Protection & Materials Licensing

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