

From: [Saba, Farideh](#)
To: [Wells, Russell Douglas](#); [Williams, Gordon Robert \(grwilliams1@tva.gov\)](#)
Cc: [Schrull, Edward Dustin \(edschrull@tva.gov\)](#); [Venkataraman, Booma](#); [Young, Austin](#)
Subject: Browns Ferry Unit 1: RAI Associated with Relief Request 1-ISI-28
Date: Tuesday, September 11, 2018 3:53:00 PM
Attachments: [RAI for Browns Ferry RR 1-ISI-28 regarding impractical weld inspections.docx](#)
Importance: High

Gordon and Russel,

By letter dated May 31, 2018 (Agencywide Documents Access and Management System (ADAMS) Accession Number ML18177A379), Tennessee Valley Authority (TVA) submitted relief requests 1-ISI-28 and -29, requesting relief from the American Society of Mechanical Engineers (ASME) Boiler and Pressure Vessel Code (Code), Section XI, Rules for Inservice Inspection of Nuclear Power Plant Components, for Browns Ferry Nuclear Plant Unit 1. Specifically, pursuant to Title 10 of the *Code of Federal Regulations* (10 CFR) 50.55a(g)(5)(iii), the licensee requested relief from the “essentially 100 percent” volumetric examination coverage requirements of ASME Code Section XI for the welds on the basis that the code requirement is impractical. The NRC has determined that the following additional information is necessary to complete its review and make a regulatory decision.

The NRC staff reviewed the licensee’s submittal and determined that additional information, as described in the attached request for additional information (RAI), is needed for the staff to complete its review of the relief request 1-ISI-28. The NRC staff forwarded by electronic mails draft RAIs to TVA. On August 30, 2018, the NRC staff held a conference call to provide the TVA with an opportunity to clarify any portion of the draft RAIs and discuss the time frame for which TVA would provide the requested information. The final RAI as discussed with TVA is attached to this email. During this call TVA proposed to submit its responses by October 15, 2018, and the NRC staff agreed with the proposed date.

Thanks,

Farideh

Farideh E. Saba, P.E.

Senior Project Manager

NRC/ADRO/NRR/DORL

301-415-1447

Mail Stop O-8B01A

Farideh.Saba@NRC.GOV

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELIEF REQUEST 1-ISI-28 REGARDING EXAMINATION COVERAGE OF
INSIDE RADIUS AND NOZZLE-TO-VESSEL WELDS
TENNESSEE VALLEY AUTHORITY
BROWNS FERRY NUCLEAR PLANT, UNIT 1
DOCKET NO. 50-259

1.0 INTRODUCTION

By letter dated May 31, 2018 (Agencywide Documents Access and Management System (ADAMS) Accession Number ML18177A379), Tennessee Valley Authority (TVA, the licensee) submitted relief requests 1-ISI-28 and -29, requesting relief from the American Society of Mechanical Engineers (ASME) Boiler and Pressure Vessel Code (Code), Section XI, Rules for Inservice Inspection of Nuclear Power Plant Components, for Browns Ferry Nuclear Plant (Browns Ferry) Unit 1. Specifically, pursuant to Title 10 of the *Code of Federal Regulations* (10 CFR) 50.55a(g)(5)(iii), the licensee requested relief from the “essentially 100 percent” volumetric examination coverage requirements of ASME Code Section XI for the welds on the basis that the code requirement is impractical. The NRC has determined that the following additional information is necessary to complete its review and make a regulatory decision.

RAI 1

How do the coverages obtained during this interval compare to those of the last interval. If there has been any drop in coverage from the last interval, please discuss what caused the decrease in obtainable coverage and justify how this examination provides an equivalent or greater standard of quality and safety.

RAI 2

Regarding weld numbers N2D-IR (Recirc Inlet), N2E-IR (Recirc Inlet), N2G-IR (Recirc Inlet), N5A-IR (Core Spray), N5B-IR (Core Spray), N8A-IR (Jet Pump Instrumentation), and N10-IR (SLC) please provide diagrams showing the coverage that was obtained, coverage calculations and obstructions inhibiting further examination.

RAI 3

Regarding the 22 indications that were found in the vicinity of Feedwater nozzle N-4A, please confirm whether there were any changes in number, or growth, of indications compared to those found in U1-R6. Additionally, if any changes were present, please confirm that Browns Ferry will follow any ASME Code required adjustments to the examination schedule.