



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

August 31, 2018

Mr. Art Wharton
Studsvik Scandpower, Inc.,
3700 O'Donnell Street, Suite 200
Baltimore, MD 21224 USA

Subject: CLARIFICATION OF SAFETY EVALUATION FOR STUDSVIK
SCANDPOWER INC., TOPICAL REPORT SSP-14-P01/028-TR, "GENERIC
APPLICATION OF THE STUDSVIK SCANDPOWER CORE MANAGEMENT
SYSTEM TO PRESSURIZED WATER REACTORS"

Dear Mr. Wharton:

By letter dated December 18, 2015 (Agencywide Documents Access and Management System Accession No. ML15355A283), Studsvik Scandpower Inc. submitted Topical Report (TR) SSP-14-P01/028-TR, "Generic Application of the Studsvik Scandpower Core Management System to Pressurized Water Reactors," to the U.S. Nuclear Regulatory Commission (NRC) staff for review and approval. By letter dated September 15, 2017 (ADAMS Accession No. ML17236A393), the NRC staff approved the TR SSP-14-P01/028-TR as acceptable for referencing in licensing applications for nuclear power plants to the extent specified and under the limitations delineated in the TR and in the enclosed final safety evaluation report (SE). This letter clarifies the NRC position on "Change Management" section of the SE.

The "Change Management" section of the SE states as follows:

Any change management with respect to addition of new features, new functionality, correction of software errors, and/or usage of additional data from operating reactors/test reactors/Halden must ensure that the nuclear reliability factors generated by exercising the methodology reported in this CMS5 TR remain conservative.

The NRC staff is providing a clarification to the above statement as below:

1. New features and new functionalities shall not include new models, new correlations, and new numerical methods of solution. For implementation of new models, new correlations, and changes in the solution methods into the approved version of the topical report shall be subject to NRC staff approval.

The above clarifications do not affect the technical meaning or findings in the original safety evaluation for the TR.

Please contact Leslie Perkins at 301-415-2375 or via e-mail at Leslie.Perkins@nrc.gov with any questions you may have regarding this letter.

Sincerely,

/RA/

Dennis C. Morey, Chief
Licensing Processes Branch
Division of Policy and Rulemaking
Office of Nuclear Reactor Regulation

Docket No. 99902035

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