

50-315

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## DESCRIPTION

Ltr. Notorized 03-21-77...Ref our 01-17-77 ltr...Trans The Following:

( 2 pages ) **DO NOT REMOVE**PLANT NAME: DONALD C. COOK UNIT # 1  
jcm**ACKNOWLEDGED**

## ENCLOSURE

Consists of Requested info. regarding Potential consequences of a fuel handling accident inside the containment of Donald C. Cook Plant Unit No. 1.....

( 4 pages )

## SAFETY

## FOR ACTION/INFORMATION

## ENVIRO

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BRANCH CHIEF:

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Ziemann (5)  
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BROOKHAVEN NAT. LAB.

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1. The first part of the document is a list of names and addresses of the members of the committee.

2. The second part of the document is a list of names and addresses of the members of the committee.

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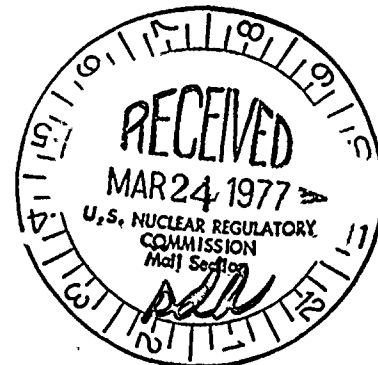
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# INDIANA & MICHIGAN POWER COMPANY

P. O. BOX 18  
BOWLING GREEN STATION  
NEW YORK, N. Y. 10004

**REGULATORY DOCKET FILE COPY** 21, 1977



Donald C. Cook Nuclear Plant Unit No. 1  
Docket No. 50-315  
DPR No. 58

Mr. Bernhard C. Rusche, Director  
Office of Nuclear Reactor Regulation  
U. S. Nuclear Regulatory Commission  
Washington, D. C. 20555

Dear Mr. Rusche:

This letter is in response to Mr. Dennis L. Ziemann's request, dated January 17, 1977, for a detailed evaluation of the potential consequences of a fuel handling accident inside the containment of Donald C. Cook Nuclear Plant Unit No. 1. The evaluation was to use assumptions comparable to those given in Regulatory Guide 1.25 "Assumptions Used For Evaluating the Potential Radiological Consequences of a Fuel Handling Accident in the Fuel Handling and Storage Facility for Boiling and Pressurized Water Reactors." It was further specified that the analysis should be performed in two parts: (1) a conservative analysis using parameters as limited by the technical specifications and (2) an analysis using parameters associated with current known facility operating conditions. The details of these analyses are presented in Attachment 1.

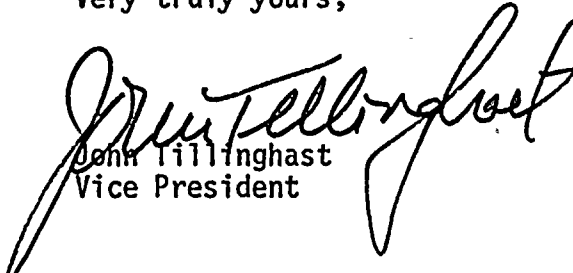
The conservative analysis based on the assumptions in Regulatory Guide 1.25 yielded results which indicated that an individual at the site boundary would receive a 0-2 hour thyroid dose of 82.3 Rem and a 0-2 hour whole body dose of 1.3 Rem. In the realistic analysis, employing the assumptions listed in Regulatory Guide 4.2 appendix I, the 0-2 hour thyroid and whole body doses to an individual at the site boundary were estimated to be  $5.61 \times 10^{-3}$  Rem and  $3.73 \times 10^{-4}$  Rem, respectively.

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Based on this analysis, the radiological consequences of a fuel handling accident inside the containment of Donald C. Cook Nuclear Plant Unit No. 1 are well within the limits (300 Rem to thyroid and 25 Rem to whole body) of 10 CFR Part 100.

Very truly yours,

  
John Lillinghast  
Vice President

Sworn and subscribed to before me  
this 21<sup>st</sup> day of March, 1977 in  
New York County, New York

  
Notary Public

DAVID G. HUME  
NOTARY PUBLIC, State of New York  
No. 31-4608113  
Qualified in New York County  
Commission Expires March 30, 1979

cc: G. Charnoff  
R. C. Callen  
P. W. Steketee  
R. J. Vollen  
R. Walsh  
R. W. Jurgensen - Bridgman  
R. S. Hunter

1. The first part of the report is a summary of the work done during the year. It is a brief statement of the results of the work, and is intended to give a general idea of the progress made.

2. The second part of the report is a detailed account of the work done during the year. It is a full and complete statement of the work, and is intended to give a detailed account of the progress made.

3. The third part of the report is a summary of the work done during the year. It is a brief statement of the results of the work, and is intended to give a general idea of the progress made.

4. The fourth part of the report is a detailed account of the work done during the year. It is a full and complete statement of the work, and is intended to give a detailed account of the progress made.

5. The fifth part of the report is a summary of the work done during the year. It is a brief statement of the results of the work, and is intended to give a general idea of the progress made.

6. The sixth part of the report is a detailed account of the work done during the year. It is a full and complete statement of the work, and is intended to give a detailed account of the progress made.

## Attachment 1

### Fuel Handling Accident Inside Containment of Donald C. Cook Nuclear Plant

Potential consequences of a fuel handling accident were evaluated using (1) the conservative assumptions listed in Regulatory Guide 1.25 and (2) the realistic assumptions given in Regulatory Guide 4.2 Appendix I.

#### I. Assumptions Used

##### A. Conservative Case

- 1) Gap activity from all rods in the assembly (204 rods) was released.
- 2) Accident occurs 100 hours after the plant shutdown
- 3) Radial peaking factor is 1.65
- 4) All the gap activity in the assembly is released and consists of 10% of iodines, 10% of all noble gases other than Kr-85, and 30% of Kr-85, of the total activity in the rods at the time of the accident.
- 5) Iodines are composed of 99.75% inorganic and 0.25% organic species.
- 6) Effective pool decontamination factors for iodines and noble gases are 100 and 1, respectively.
- 7) Activity is released into the atmosphere
- 8) No plateout of iodine inside containment or in ducts
- 9) Atmospheric dilution factor ( $X/Q$  value) =  $3.15 \times 10^{-4}$  sec/ $m^3$ ; corresponds to 8-24 hour value given in Regulatory Guide 1.4.
- 10) Effective energies for Beta and gamma radiation are estimated using references given in Regulatory Guide 1.25
- 11) Breathing rates and dose conversion factors for iodines are taken from Regulatory Guide 1.25.

THE UNITED STATES OF AMERICA  
DEPARTMENT OF JUSTICE

INVESTIGATION OF THE ACTS OF VIOLENCE  
COMMITTED BY THE ORGANIZATION OF  
THE ARAB BOYCOTT (O.A.B.)

REPORT OF THE

JOINT COMMITTEE ON INVESTIGATION

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JOINT COMMITTEE ON INVESTIGATION



B. Realistic Case

- 1) Gap activity consists of 1% of the rod activity.
- 2) The gap activity from 1 row of rods in the assembly was released.
- 3) The accident occurred one week after shutdown.
- 4) Effective pool decontamination factor for iodine is 500.
- 5) X/Q value is 1/10th of that in Regulatory Guide 1.4 ( $3.15 \times 10^{-5}$  sec./m<sup>3</sup>)

II. Analytical Methods

A. Thyroid Dose Model

0-2 Hr. thyroid dose at the site boundary was estimated using  $D = \frac{Fg I_i F P R_i (X/Q) B}{(DF_p) (DF_f)}$

D = thyroid dose from Isotope I<sub>i</sub> (rads)

Fg = fraction of core damaged

I<sub>i</sub> = core iodine inventory (curies)

R<sub>i</sub> = Dose Conversion Factor for I<sub>i</sub>

P = fuel peaking factor (1.65)

B = breathing rate ( $3.4 \times 10^{-4}$  m<sup>3</sup>/sec.)

X/Q = atmospheric dilution factor (sec./m<sup>3</sup>)

DF<sub>p</sub> = pool decontamination factors

DF<sub>f</sub> = filter decontamination factor (1 for both cases - no credit is taken).

F = fraction of fuel rod iodine inventory in fuel rod void space (0.1)

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$\Delta V = \Delta V_{\text{induction}} + \Delta V_{\text{static}}$

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in Prozent zu nehmen).

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B. Whole Body Dose Equation

0-2 hour whole body dose was estimated using

$$D = (0.23 \bar{E}_\beta + 0.25 \bar{E}_\gamma) X$$

where  $\bar{E}_\beta$  = effective beta energy (Mev/dis)

$\bar{E}_\gamma$  = effective gamma energy (Mev/dis)

X = concentration of the isotope in the cloud  
(curies/m<sup>3</sup>)

III. Results

The results of this analysis are listed below:

0-2 hour site boundary doses (rem)

	Thyroid	Whole Body
10 CFR 100 Limit	300	25
Conservative Case	82.3	1.3
Realistic Case	$5.61 \times 10^{-3}$	$3.73 \times 10^{-4}$

For both conservative and realistic cases, no credit was taken for the closure of the containment isolation valve, since the radiation monitor that generates the signal for the closure of this valve is Seismic Class III and is not redundant. Further, it was observed that the minimum isolation valve closure times as specified in technical specification limits were longer than the calculated transit time for the radioactive puff to reach the valve from the pool. However, assuming no earthquake or single active failure of radiation monitor and with actual measured valve closure times as opposed to technical specification minimum, it was shown that only a small fraction of the above cited activity will be released to the environment since the isolation valve will close before the major portion of the gaseous release reaches the isolation valve.

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$$C = \left( C_{ij}^{\alpha\beta} \right) = \begin{pmatrix} C_{11}^{11} & C_{11}^{12} & C_{11}^{13} \\ C_{11}^{21} & C_{11}^{22} & C_{11}^{23} \\ C_{11}^{31} & C_{11}^{32} & C_{11}^{33} \end{pmatrix}$$

1. The first step is to identify the problem or goal. This involves understanding the current situation, identifying the problem, and setting a clear goal.

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IV. Conclusions

From the results of this analysis it is concluded that the radiological consequences due to a fuel handling accident inside containment are well within the limits set forth in 10 CFR Part 100.

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