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INCIDENT REPORTTO:
J.G. KepplerFROM: Indiana & Michigan Power Co.
Bridgman, Michigan
R.W. JurgensenDATE OF DOCUMENT
2-18-76DATE RECEIVED
2-3-76☐ LETTER
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☒ UNCLASSIFIED

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30

DESCRIPTION

Ltr. trans the following.....

ENCLOSURE

R/O # 76-04, on 1-20-76, Concerning Potential
for fuel clad stress cracking.....

(30 Cys, Received)

PLANT NAME: Cook # 1

DO NOT REMOVE
ACKNOWLEDGEDNOTE: IF PERSONNEL EXPOSURE IS INVOLVED
SEND DIRECTLY TO KREGER/J. COLLINS

SAFETY

FOR ACTION/INFORMATION

ENVIRO

SAB 2-24-76

BRANCH CHIEF:

Ziemann

W/3 CYS FOR ACTION

LIC. ASST:

Diggs

W/ CYS

ACRS 16 CYS ~~XXXXXX~~ SENT TO LA

INTERNAL DISTRIBUTION

REG FILE

NRC PDR

I & E (2)

MIPC (3)

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KREGER/J. COLLINS

EXTERNAL DISTRIBUTION

LPDR: St. Joseph, Michigan

TIC

NSIC

CONTROL NUMBER

1770

Regulatory

File Cy

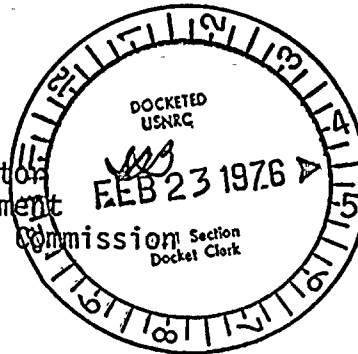


INDIANA & MICHIGAN POWER COMPANY

DONALD C. COOK NUCLEAR PLANT
P.O. Box 458, Bridgman, Michigan 49106

February 18, 1976

Mr. J.G. Keppler, Regional Director
Office of Inspection and Enforcement
United States Nuclear Regulatory Commission
Region III
799 Roosevelt Road
Glen Ellyn, MI 60137



Operating License DPR-58
Docket No. 50-315

Dear Mr. Keppler:

Attached for your information is Reportable Occurrence:

RO-50-315/76-04

Nuclear Regulatory Commission Regulatory Guide 1.16, Revision 3, Section 2.a.9 requires that we report the "discovery during plant life of conditions not specifically considered in the Safety Analysis Report or Technical Specifications that require remedial action or corrective measures to prevent the existence of an unsafe condition".

Mr. K.R. Baker, NRC - RO: III inspector, was informed of this event on January 22, 1976. Since that time this problem has been discussed with the NRC Staff and at our recent ACRS hearing on our full power license application.

Section 2.a.9 of Regulatory Guide 1.16, Revision 3, would normally require a 24 hour written followup to our report to Mr. Baker. However, we do not know whether any action will eventually be required as a result of this problem. Because of this we are filing this report as a Thirty Day Written Report since this problem has "lesser immediate importance" than those described under the Twenty-Four Hour Written Report category.

Sincerely,

R.W. Jurgensen
Plant Manager

/bab

cc: Listed on following page.

1770

12

cc: R.S. Hunter
J.E. Dolan
G.E. Lien
R. Kilburn
R.J. Vollen BPI
R.C. Callen MPSC
K.R. Baker RO: III
P.W. Steketee, Esq.
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G. Olson
J.M. Hennigan
PNSRC
R.S. Keith
Dir., IE (30 copies)
Dir., MIPC:(3 copies)

LICENSEE EVENT REPORT

CONTROL BLOCK:

[PLEASE PRINT ALL REQUIRED INFORMATION]

LICENSEE NAME										LICENSE NUMBER										LICENSE TYPE					EVENT TYPE	
01	M	I	D	C	C	1	0	0	-	0	0	0	0	0	-	0	0	4	1	1	1	1	0	3		
7	8	9				14	15										25	26				30	31	32		

CATEGORY		REPORT TYPE	REPORT SOURCE	DOCKET NUMBER								EVENT DATE					REPORT DATE								
01	CONT	Z	Z	L	L	0	5	0	-	0	3	1	5	0	1	2	0	7	6	0	2	1	8	7	6
57	58	59	60			61							68	69				74	75					80	

EVENT DESCRIPTION

02	Potential for fuel clad stress cracking - See Attachment.	80
03		80
04		80
05		80
06	RO 50-315/76-04	80

7 8 9 SYSTEM CODE CAUSE CODE COMPONENT CODE PRIME COMPONENT SUPPLIER COMPONENT MANUFACTURER VIOLATION

07 R C F F U E L X X N W 1 2 0 N

7 8 9 10 11 12 17 43 44 47 48

CAUSE DESCRIPTION

08	NA	80
09		80
10		80

	FACILITY STATUS	% POWER	OTHER STATUS	METHOD OF DISCOVERY	DISCOVERY DESCRIPTION
[1][1]	[E]	[0][8][1]	[NA]	[d]	[NA]
<small>7 8</small>	<small>9</small>	<small>10 12</small>	<small>13 44</small>	<small>45</small>	<small>46 80</small>

FORM OF ACTIVITY RELEASED

CONTENT OF RELEASE

AMOUNT OF ACTIVITY

LOCATION OF RELEASE

PERSONNEL EXPOSURES

NUMBER			TYPE	DESCRIPTION
13	000	Z	NA	

PERSONNEL INJURIES

NUMBER				DESCRIPTION	
1	4	0	0	0	NA

7 8 9 11 12 80

PROBABLE CONSEQUENCES

[illegible]

LOSS OR DAMAGE TO FACILITY

TYPE			DESCRIPTION
16	Z		NA

PUBLICITY

17 | NA

ADDITIONAL FACTORS

[illegible]

7 8 9 80

NAME: R.W. Jurgensen

PHONE: 465-5901

Supplement to R0 50-315/76-04 Event Description

On January 22, 1976, we informed Mr. K.R. Baker, NRC Region III Inspector, that representatives of Westinghouse Electric Corporation had informed the American Electric Power Service Corporation that Westinghouse had been conducting an evaluation of fuel pellet-clad interaction and the potential for fuel clad failures in high power density (6.7 Kw/foot) reactors due to this phenomena.

Subsequently, on February 5, 1976, this subject was discussed during the 190th General Meeting of the Advisory Committee on Reactor Safeguards in connection with the request for a full power license for the Donald C. Cook Nuclear Plant Unit 1. At that meeting, we reported that Westinghouse was attempting to correlate pellet clad interaction data from test reactors and test fuel with commercial reactor operating conditions, but that no failures due to pellet-clad interaction in Westinghouse commercial reactors in normal operation or under ANSI N18.2 Condition II transients had been experienced.

Furthermore, we reported that the recent Westinghouse correlation indicates that there should be no fuel failures in Donald C. Cook Nuclear Plant Unit 1 at 100 percent power during normal operation or during the ANSI N18.2 Condition II transients.

This subject will receive continuing investigation by the NRC Staff and by Westinghouse. We will keep abreast of this work.