

50-315

## NRC DISTRIBUTION FOR PART 50 DOCKET MATERIAL

FILE NUMBER  
INCIDENT REPORT

TO: J G Keppler

FROM: Indiana & Michigan Pwr Co  
Bridgman, Michigan  
R W JurgensenDATE OF DOCUMENT  
2-2-77DATE RECEIVED  
2-15-77☒ LETTER  
☒ ORIGINAL  
☐ COPY  
☐ NOTORIZED  
☒ UNCLASSIFIED

PROP

INPUT FORM

NUMBER OF COPIES RECEIVED  
one signed

## DESCRIPTION

Ltr trans the following:

1p

PLANT NAME: Cook #1

## ENCLOSURE

Licensee Event Report (RO# 77-03) on 1-19-77  
concerning removal of residual heat removal  
loop in excessive of time allowed per tech  
specs due to personnel error.....

1p

ACKNOWLEDGED  
DO NOT REMOVENOTE: IF PERSONNEL EXPOSURE IS INVOLVED  
SEND DIRECTLY TO KREGER/J. COLLINS

## FOR ACTION/INFORMATION

2-15-77

ehf

BRANCH CHIEF:

Ziemann

W/3 CYS FOR ACTION

LIC. ASST.:

D. SSS

W/1 CYS

ACRS - 16 CYS HOLDING/SENT - 45 - CAT B 2/15/77

## INTERNAL DISTRIBUTION

REG-FILE

NRC PDR

I &amp; E (2)

MIPC

SCHROEDER/IPPOLITO

HOUSTON

NOVAK/CHECK

GRIMES

CASE

BUTLER

HANAUER

TEDESCO/MACCARY

EISENHUT

BAER

SHAO

VOLLMER/BUNCH

KREGER/J. COLLINS

## EXTERNAL DISTRIBUTION

LPDR: ST JOSEPH MI

TIC:

NSIC:

## CONTROL NUMBER

1601

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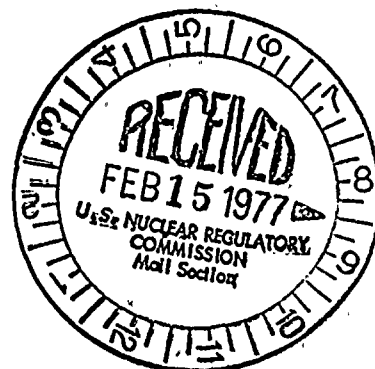


**INDIANA & MICHIGAN POWER COMPANY**

DONALD C. COOK NUCLEAR PLANT  
P.O. Box 458, Bridgman, Michigan 49106

*D. LANHAM*

February 2, 1977



Mr. J. G. Keppler, Regional Director  
Office of Inspection and Enforcement  
United States Nuclear Regulatory Commission  
Region III  
799 Roosevelt Road  
Glen Ellyn, IL 60137

Operating License DPR-58  
Docket No. 50-315

Dear Mr. Keppler:

Pursuant to the requirements of Appendix A Technical Specifications and the United States Nuclear Regulatory Commission Regulatory Guide 1.16, Revision 4, Section 2.a, the following report is submitted:

RO 50-315/77-03

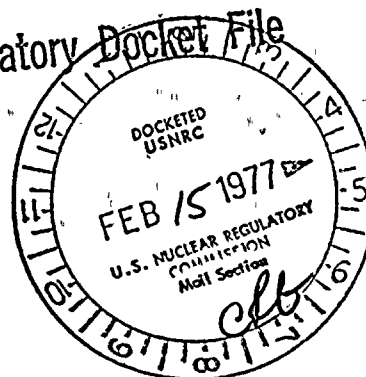
Sincerely,

*R. W. Jurgensen*  
R. W. Jurgensen  
Plant Manager

RWJ/mj

cc: R. S. Hunter  
J. E. Dolan  
G. E. Lien  
R. Kilburn  
R. J. Vollen BPI  
R. C. Callen MPSC  
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Regulatory Docket File



1601

FEB 3 1977



# LICENSEE EVENT REPORT

**CONTROL BLOCK:**

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1                          6

**[PLEASE PRINT ALL REQUIRED INFORMATION]**

LICENSEE NAME				LICENSE NUMBER												LICENSE TYPE				EVENT TYPE				
01	M	I	D	C	C	1	0	0	-	0	0	0	0	0	-	0	0	4	1	1	1	1	0	1
7	8	9				14	15										25	26					31	32

CATEGORY		REPORT TYPE	REPORT SOURCE	DOCKET NUMBER						EVENT DATE				REPORT DATE									
01	CON'T			0	5	0	-	0	3	1	5	0	1	1	9	7	7	0	2	0	1	7	7
7	8	57	58	59	60	61					68	69				74	75						80

**EVENT DESCRIPTION**

02	THE INSERVICE RESIDUAL HEAT REMOVAL (RHR) LOOP WAS REMOVED FROM SERVICE FOR 2 HOURS	80
03	AND 10 MINUTES IN VIOLATION OF TECHNICAL SPECIFICATION 3.9.8 ACTION ITEM (b) WHICH	80
04	PERMITS REMOVAL OF THE RHR LOOP FOR UP TO 1 HOUR PER 8 HOUR PERIOD DURING PERFORMANCE	80
05	OF CORE ALTERATIONS. CORE ALTERATIONS WERE IN PROGRESS.	80
06	(LER-R0-50-315/77- 03 )	80

SYSTEM CODE		CAUSE CODE	COMPONENT CODE					COMPONENT SUPPLIER	COMPONENT MANUFACTURER				VIOLATION		
0	7	C	F	A	Z	Z	Z	Z	Z	Z	Z	Z	Z	Y	
7	8	9	10	11	12				17		43			47	48

**CAUSE DESCRIPTION**

08	THE OPERATOR LOST TRACK OF THE TIME THAT HAD PASSED SINCE THE RHR LOOP HAD BEEN	80
09	REMOVED FROM SERVICE.	80
10		80

		FACILITY STATUS	% POWER	OTHER STATUS	METHOD OF DISCOVERY	DISCOVERY DESCRIPTION	
7	8		10	12	13	44	80
1	1	H	0	0	N.A.	A	N.A.

		FORM OF ACTIVITY RELEASED	CONTENT OF RELEASE	AMOUNT OF ACTIVITY	LOCATION OF RELEASE
7	8		10	11	44
1	2	Z	Z	N.A.	N.A.

## PERSONNEL EXPOSURES

NUMBER				TYPE	DESCRIPTION
13	0	0	0	Z	N.A.

## PERSONNEL INJURIES

NUMBER				DESCRIPTION
1	4	0	0	N.A.

### PROBABLE CONSEQUENCES

1	5	NONE	
7	8	9	80

**LOSS OR DAMAGE TO FACILITY**

TYPE			DESCRIPTION
1	6	Z	N.A.

## PUBLICITY

17	N.A.	80
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### ADDITIONAL FACTORS

18 CONTROL OF THIS OPERATION IS NOW UNDER THE DIRECT SUPERVISION OF THE SENIOR REACTOR 80

19 OPERATOR IN CHARGE OF CORE ALTERATIONS. 80

NAME: Al Jackson PHONE: (616) 465-5901

GPO 881-867

