

50-315

NRC DISTRIBUTION FOR PART 50 DOCKET MATERIAL

FILE NUMBER
INCIDENT REPORT

TO: Mr. J. G. Keppler

FROM: Indiana & Mich Power Co
Bridgman, Mich 49106
R.W. JurgensenDATE OF DOCUMENT
3-21-77DATE RECEIVED
3-12-77☒ LETTER
☒ ORIGINAL
☐ COPY☐ NOTORIZED
☒ UNCLASSIFIED

PROP

INPUT FORM

NUMBER OF COPIES RECEIVED

1 SIGNED

DESCRIPTION Ltr trans the following: 1P

ENCLOSURE Lic Event Report 77-10 occurring
on 2-22-77 regarding three inoperable safety
valves on the steam generator due to failure by the
the shift operating engineer... 2P

(1 cy encl rec'd)

PLANT NAME: Cook Unit 1

DHL

ACKNOWLEDGED
DO NOT REMOVENOTE: IF PERSONNEL EXPOSURE IS INVOLVED
SEND DIRECTLY TO KREGER/J. COLLINS

FOR ACTION/INFORMATION

BRANCH CHIEF:	Ziemann
W/3 CYS FOR ACTION	
LIC. ASST.:	Diggs
W/1 CYS	
ACRS 16 CYS HOLDING/SENT	As CAT B

INTERNAL DISTRIBUTION

REG FILE				
NRC PDR				
I & E (2)				
MIPC				
SCHROEDER/IPPOLITO				
HOUSTON				
NOVAK/CHECK				
GRIMES				
CASE				
BUTLER				
HANAUER				
TEDESCO/MACCARY				
EISENHUT				
BAER				
SHAO				
VOLLMER/BUNCH				
KREGER/J. COLLINS				

EXTERNAL DISTRIBUTION

LPDR: St Joseph, Mi					
TIC:					
NSIC:					

CONTROL NUMBER

771050229

60

55-1-8

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INDIANA & MICHIGAN POWER COMPANY

DONALD C. COOK NUCLEAR PLANT
P.O. Box 458, Bridgman, Michigan 49106

March 21, 1977

Mr. J. G. Keppler, Regional Director
Office of Inspection and Enforcement
United States Nuclear Regulatory Commission
Region III
799 Roosevelt Road
Glen Ellyn, IL 60137



Regulatory
Operating License DPR-58
Docket No. 50-315

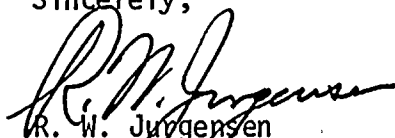
File *cy*

Dear Mr. Keppler:

Pursuant to the requirements of Appendix A Technical Specifications and the United States Nuclear Regulatory Commission Regulatory Guide 1.16, Revision 4, Section 2.b, the following report is submitted:

RO-50-315/77-10

Sincerely,


R. W. Jorgensen
Plant Manager

RWJ/mj

cc: R. S. Hunter
J. E. Dolan
G. E. Lien
R. Kilburn
R. J. Vollen BPI
R. C. Callen MPSC
K. R. Baker RO: III
R. Walsh, Esq.
P. W. Steketee, Esq.
G. Charnoff, Esq.
G. Olson
J. M. Hennigan
PNSRC
R. S. Keith
Dir., IE (30 copies)
Dir., MIPC (3 copies)

MAR 23 1977

771050230

LICENSEE EVENT REPORT

CONTROL BLOCK:

[PLEASE PRINT ALL REQUIRED INFORMATION]

LICENSEE NAME	LICENSE NUMBER	LICENSE TYPE	EVENT TYPE
01 M I D C C 1	00-000000-00	41111	03
7 8 9 14	15 25	26 30	31 32

CATEGORY	REPORT TYPE	REPORT SOURCE	DOCKET NUMBER	EVENT DATE	REPORT DATE
01 CONT	L	L	050-0315	022277	031877
7 8 57 58	59 60	61 68	69 74	75 80	

EVENT DESCRIPTION

02 WITH THE REACTOR AT 10% OF RATED THERMAL POWER (RTP) AND THREE INOPERABLE STEAM LINE

03 SAFETY VALVES ON AN OPERATING STEAM GENERATOR THE POWER RANGE NEUTRON FLUX HIGH SET-

04 POINT WAS INADVERTANTLY RAISED TO ITS NORMAL VALUE OF 109% OF RTP. WITH THREE

05 INOPERABLE SAFETY VALVES THE MAXIMUM ALLOWABLE POWER RANGE NEUTRON FLUX HIGH SETPOINT

06 IS 43.6% OF RTP PER TECHNICAL SPECIFICATION. 3.7.1.1.(SEE SUPPLEMENT) (RO-50-315/77-10)

SYSTEM CODE	CAUSE CODE	COMPONENT CODE	PRIME COMPONENT SUPPLIER	COMPONENT MANUFACTURER	VIOLATION
07 Z Z	A	Z Z Z Z Z	Z	Z 9 9 9	N
7 8 9 10	11	12 17	43	44 47	48

CAUSE DESCRIPTION

08 THE SHIFT OPERATING ENGINEER GRANTED PERMISSION TO PERFORM INSTRUMENT SURVEILLANCE

09 AND TO RAISE THE NEUTRON FLUX HIGH SETPOINT ON THE POWER RANGE CHANNELS TO PERMIT

10 POWER ESCALATION TO 30% OF RTP. THE TRIP SETPOINT WAS RAISED (SEE SUPPLEMENT)

FACILITY STATUS	% POWER	OTHER STATUS	METHOD OF DISCOVERY	DISCOVERY DESCRIPTION
11	010	NA	A	NA
7 8 9	10 12	13	44 45	46 80

FORM OF ACTIVITY RELEASED	CONTENT OF RELEASE	AMOUNT OF ACTIVITY	LOCATION OF RELEASE
12 Z	Z	NA	NA
7 8 9	10 11	44	45 80

PERSONNEL EXPOSURES

NUMBER	TYPE	DESCRIPTION
13 000	Z	NA
7 8 9 11	12	13 80

PERSONNEL INJURIES

NUMBER	DESCRIPTION
14 000	NA
7 8 9 11	12 80

PROBABEE CONSEQUENCES

15 NA

LOSS OR DAMAGE TO FACILITY

TYPE	DESCRIPTION
16 Z	NA
7 8 9 10	80

PUBLICITY

17

ADDITIONAL FACTORS

18 NA

19 NA

NAME:

B. J. Svensson

PHONE: (616) 465-5901 Ext. 313

EVENT DESCRIPTION

THE ERROR WAS REALIZED IN APPROXIMATELY 2 MINUTES AND THE HIGH FLUX TRIP SETPOINT IMMEDIATELY DECREASED TO 40% OF RTP.

CAUSE DESCRIPTION

TO ITS NORMAL VALUE, 109% OF RTP, DUE TO FAILURE BY THE SHIFT OPERATING ENGINEER TO ASCERTAIN WHAT VALUE THE TRIP SETPOINT WAS BEING RAISED TO, ASSUMING THAT IT WAS BELOW 43.6% OF RTP.

