

B 04/10/78

REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)
DISTRIBUTION FOR INCOMING MATERIAL

50-315

REC: KEPPLER J G
NRC

ORG: SHALLER D V
IN & MI PWR

DOCDATE: 04/04/78
DATE RCVD: 04/10/78

DOCTYPE: LETTER NOTARIZED: NO
SUBJECT:

COPIES RECEIVED
LTR 1 ENCL 1

LICENSEE EVENT REPT (RO 50-315/78-000) ON 01/07/78 CONCERNING REACTOR PWR ON
UNIT 1 REDUCED SLOWLY TO 8% PWR AND THE REACTOR TRIPPED FROM 8%... W/ATT
SUPPORTING INFO.

PLANT NAME: COOK - UNIT 1

REVIEWER INITIAL: XJM
DISTRIBUTOR INITIAL: *wil*

***** DISTRIBUTION OF THIS MATERIAL IS AS FOLLOWS *****

INCIDENT REPORTS
(DISTRIBUTION CODE A002)

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KNIGHT**W/ENCL
HANAUER**W/ENCL
EISENHUT**W/ENCL
SHAO**W/ENCL
KREGER/J. COLLINS**W/ENCL
K SEYFRIT/IE**W/ENCL

NRC PDR**W/ENCL
MIPC**W/3 ENCL
HOUSTON**W/ENCL
GRIMES**W/ENCL
BUTLER**W/ENCL
TEDESCO**W/ENCL
BAER**W/ENCL
VOLLMER/BUNCH**W/ENCL
ROSA**W/ENCL

EXTERNAL: LPDR'S
ST. JOSEPH, MI**W/ENCL
TIC**W/ENCL
NSIC**W/ENCL
ACRS CAT B**W/16 ENCL

COPIES NOT SUBMITTED PER
REGULATORY GUIDE 10.1

DISTRIBUTION: LTR 45 ENCL 45
SIZE: 2P+1P+9P

CONTROL NBR: 7810100474
60

***** THE END *****



INDIANA & MICHIGAN POWER COMPANY

DONALD C. COOK NUCLEAR PLANT
P.O. Box 458, Bridgman, Michigan 49106
(616) 465-5901

REGULATORY DOCKET FILE COPY

April 4, 1978

Operating License DPR-58
Docket # 50-315

1978 APR 10 PM 4 39

RECEIVED DISTRIBUTION
SERVICES UNIT

US NRC
DISTRIBUTION SERVICES
BRANCH

Mr. J. G. Keppler, Regional Director
Office of Inspection and Enforcement
U. S. Nuclear Regulatory Commission
Region III
799 Roosevelt Road
Glen Ellyn, Ill. 60137

Dear Mr. Keppler:

This report is submitted pursuant to the requirement of Appendix A Technical Specifications 3.4.8 and 6.9.1. On January 7, 1978 the dose equivalent iodine-131 activity was found out of specification.

At 1400 on January 7, 1978 reactor power on Unit 1 was reduced slowly to 8% power and the reactor tripped from 8%. Power reduction was done for a scheduled repair/investigation outage.

Routine surveillance at 2100 January 7, 1978 showed that the dose equivalent iodine-131 had spiked with a maximum value noted of 1.1 μ Ci/gram.*

Reactor power had been varied considerably the previous week with no evidence of iodine spiking noted. During power reduction CVCS purification flow through the mixed bed demineralizer was approximately 75 gpm, as it had been for the majority of the week. No degassing operations were associated with this occurrence.

Analysis of the reactor coolant prior to this excursion had shown dose equivalent iodine-131 to range from $9.12 \times 10^{-3} \mu$ Ci/g to $2.80 \times 10^{-2} \mu$ Ci/g during the period of numerous power changes. Analysis at 0500 on 1-8-78 showed the iodine-131 had decreased to 0.539 μ Ci/g with dose equivalent iodine-131 at 0.675 μ Ci/g. This activity continued to decrease even during power ascension on 1-9-78. Once stable at 100% dose equivalent iodine-131 remained constant at approximately $2.3 \times 10^{-2} \mu$ Ci/g.

Iodine release at this time period is consistent with data reported in Westinghouse Electric Corporation WCAP-8637 "Iodine Behavior under transient conditions in the Pressurized Water Reactor". Dose equivalent iodine-131 values were in the "Acceptable Operation" portion of Technical

781010047

4002
5/11

April 4, 1978
Mr. J. G. Keppler
Page 2

Specification Figure 3.4.1 at all times during the transient. Fuel burnup by core region is indicated in the attached table.

This report does not meet the 30 day Technical Specification reporting requirement. This is presently under investigation and a Licensee Event Report will be submitted describing the findings and resolutions.

* Coolant samples are brought to ambient conditions before counting; therefore, units of $\mu\text{Ci}/\text{gram}$ and $\mu\text{Ci}/\text{cc}$ are interchangeable.

Very truly yours,



D. V. Shaller
Plant Manager

ms

attachments

cc: R. W. Jurgensen
J. E. Dolan
R. Kilburn
R. F. Kroeger
R. J. Vollen, BPI
K. R. Baker, RO:III
P. W. Steketee, Esq.
R. C. Callan
R. Walsh, Esq.
G. Olson
J. Stietzel
PNSRC File
G. Charnoff, Esq.
J. M. Hennigan
Dir, IE (20 copies)
Dir, MIPC (2 copies)

CORE REGION

4 (D)

2 (B)

3 (C)

BURNUP FOR
PERIOD (MWD/MTU)
10-1-77 to 1-18-78

0.7672×10^4

0.2622×10^5

0.2192×10^5

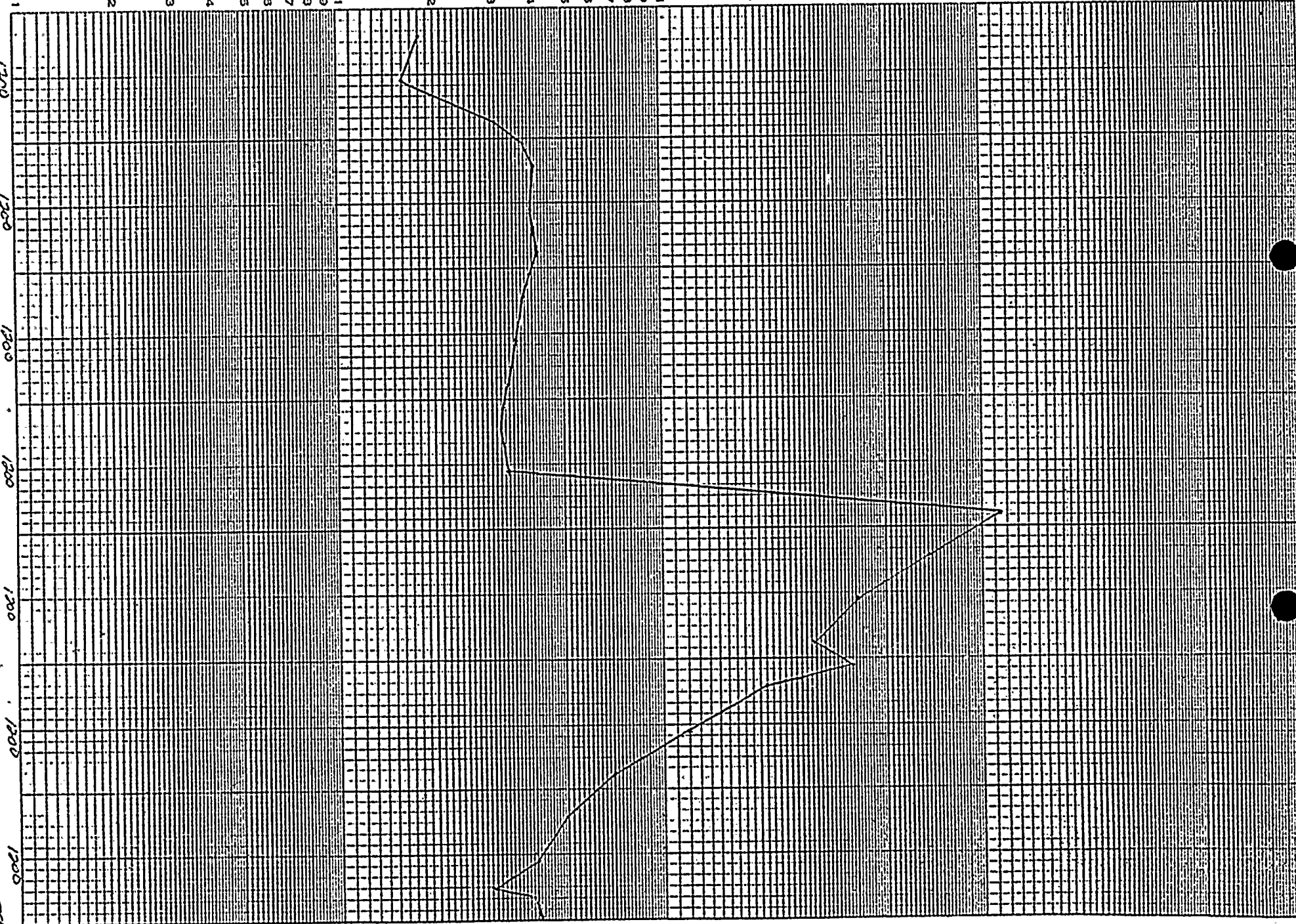
10569
131
10-11

10-31

10-31

1-4-78 1200
1-5-78 1200
1-6-78 1200
1-7-78 1200
1-8-78 1200
1-9-78 1200
1-10-78 1200

DC Base FLAUT



131
10⁻¹

10⁻³

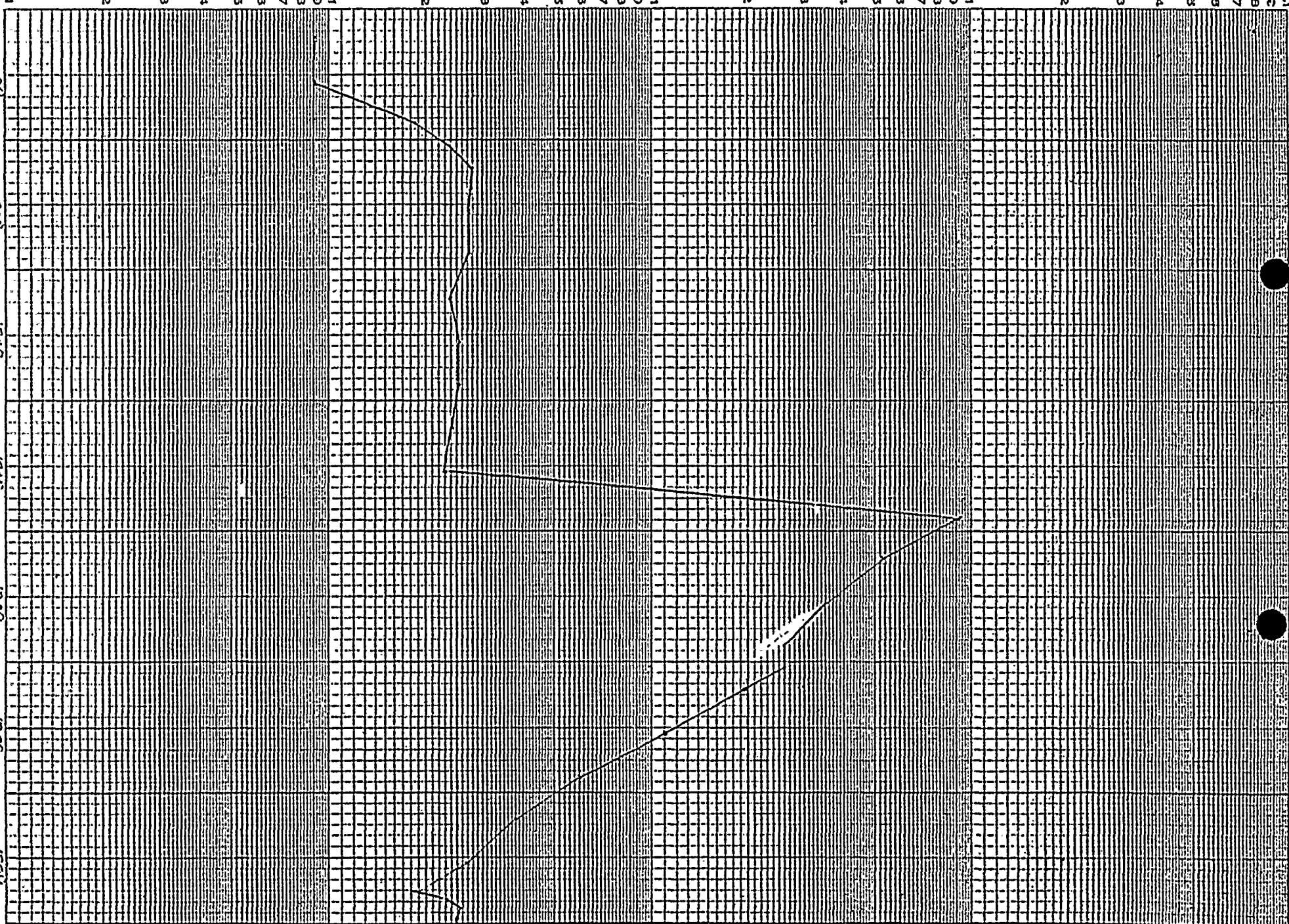
10⁻²

10⁻¹

10⁰

DC Cook. PLANT

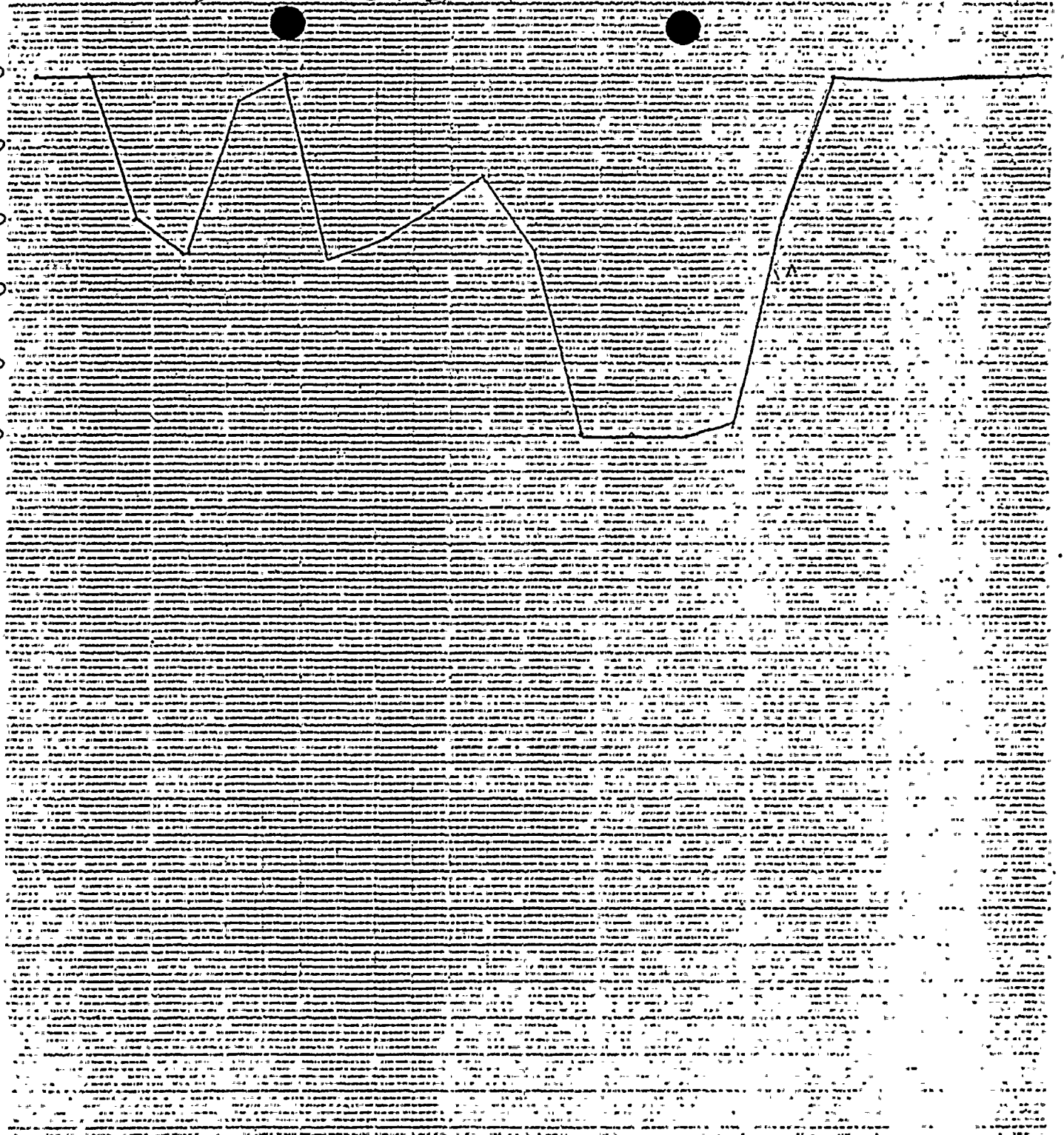
1-4-78 1200
1-5-78 1200
1-6-78 1200
1-7-78 1200
1-8-78 1200
1-9-78 1200
1-10-78 1200



COOK PLANT

2x
or
76)

100
80
60
40
20
0



Letdown Flow (gpm)

30
20
10
0

1-4-78 1-5-78 1-6-78 1-7-78 1-8-78 1-9-78 1-10-78

**DONALD C. COOK
NUCLEAR PLANT
CONDITION REPORT**

(1) CAR No. 1-1-78-13
(2) Date 1-9-78 Time 1200
(3) Category A B C D E
(4) Classified By SES/HB
(5) AO/AEO No. _____

TO: Plant Manager

LER DUE BY 1/30/78

Special Report

(6) Item Reported On Dose Equivalent
I-131 in the Unit 1 R.C.S. out of
specification

(7) Plant Conditions
Unit #1 Mode 2 Testing () Yes (☒) No
Unit #2 Mode _____ Testing () Yes () No

(8) Description of Condition At 2100 on 1-7-78, the dose equivalent I-131 on
the reactor coolant system was found to be 1.14 μ Ci/cc. This was following
unit 1 trip from ~8% power at 1426

() Additional Pages

Reported By *James W. Wick*

(9) Corrective Action Taken

By _____

(10) Off-Site Notification Made By X (Complete Telephone Records & Attach)
() AEPSC () I & M () NRC () Michigan (☒) Not Applicable

(11) Investigation Assigned To E. A. SMARRELLA

(12) Investigation Report Attached report on the dose equivalent I-131
closes out the investigation

() Additional Pages

Completed By *James W. Wick*

(13) Preventive Action Taken

SEE ATTACHED.

() Additional Pages

Dept. Head Approval *E. A. Smarella*

(14) PNSRC Review: Date _____
Comments: _____

(15) Distribution:

Iodine Spike Following Power Transient - January 7, 1978

This report is submitted pursuant to the requirement of Appendix A Technical Specifications 3.4.8 and 6.9.1. On January 7, 1978 the dose equivalent iodine-131 activity was found out of specification.

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Analysis of the reactor coolant prior to this excursion had shown dose equivalent iodine-131 to range from 9.12×10^{-3} μ Ci/g to 2.80×10^{-2} μ Ci/g during the period of numerous power changes. Analysis at 0500 on 1-8-78 showed the iodine-131 had decreased to 0.539 μ Ci/g with dose equivalent iodine-131 at 0.675 μ Ci/g. This activity continued to decrease even during power ascension on 1-9-78. Once stable at 100% dose equivalent iodine-131 remained constant at approximately 2.3×10^{-2} μ Ci/g.

Iodine release at this time period is consistent with data reported in Westinghouse Electric Corporation WCAP-8637 "Iodine Behavior under transient conditions in the Pressurized Water Reactor". Dose equivalent iodine-131 values were in the "Acceptable Operation" portion of Technical Specification Figure 3.4.1 at all times during the transient. Fuel burnup by core region is indicated in the attached table.

* Coolant samples are brought to ambient conditions before counting; therefore, units of μ Ci/gram and μ Ci/cc are interchangeable.

CORE REGION

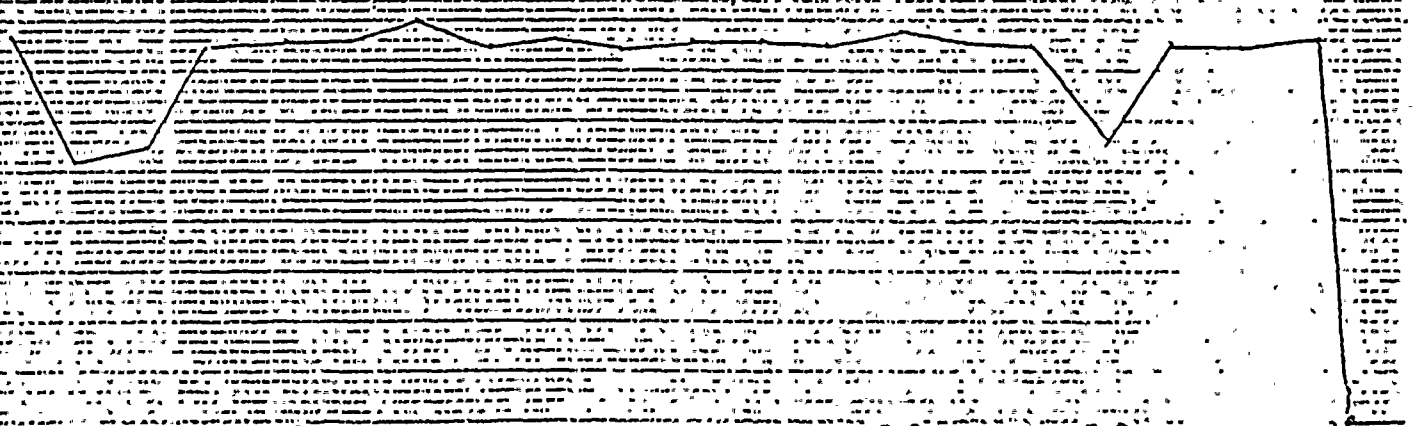
BURNUP FOR
PERIOD (MWD/MTU)
10-1-77 to 1-18-78

4 (D)	0.7672×10^4
2 (B)	0.2622×10^5
3 (C)	0.2192×10^5

100
80
60
40
20
0



30
60
40
20
0

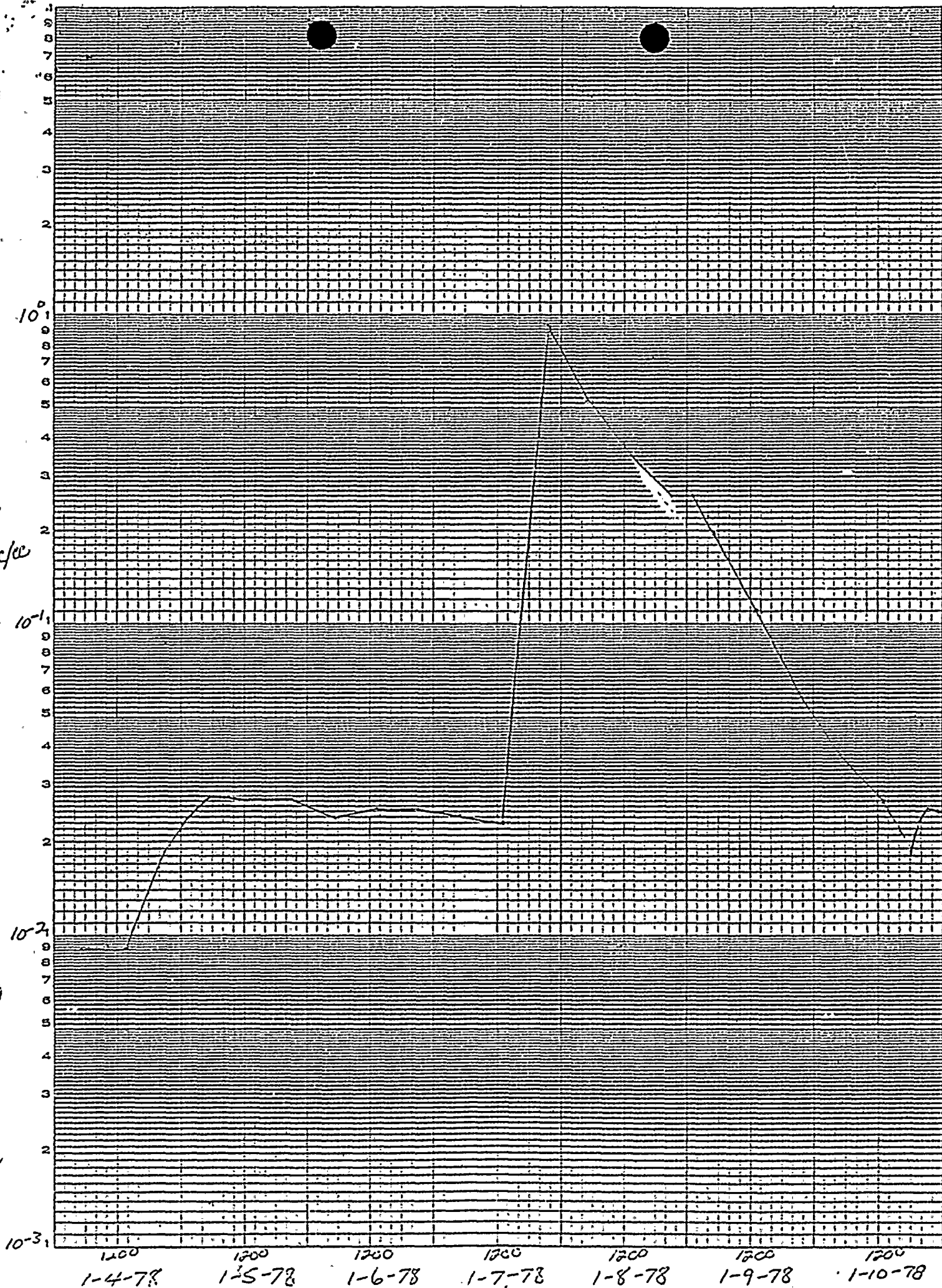


Letdown Flow (gpm)

MADE IN U. S. A.

131
11/10/78

SEMI-LOGARITHMIC
4 CYCLES X 12 DIVISIONS PER INCH



MADE IN U. S. A.

55EQ
31

SEMI-LOGARITHMIC
4 CYCLES X 12 DIVISIONS PER INCH

10⁰
10⁻¹
10⁻²
10⁻³

