

A 06/26/78

REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)
DISTRIBUTION FOR INCOMING MATERIAL

50-316

REC: KEPPLER J G
NRC

ORG: SHALLER D V
IN & MI PWR

DOCDATE: 06/19/78
DATE RCVD: 06/23/78

DOCTYPE: LETTER NOTARIZED: NO

COPIES RECEIVED

SUBJECT:

LTR 1 ENCL 1

FORWARDING LICENSEE EVENT REPT (RO 50-316/78-034) ON 05/19/78 CONCERNING
WHILE IN MODE 1 OPERATION REACTOR COOLANT SYSTEM UNIDENTIFIED LEAKAGE
EXCEEDED TECH SPEC 3. 4. 6. 2. B LIMIT... W/ATT.

PLANT NAME: COOK - UNIT 2

REVIEWER INITIAL: XJM

DISTRIBUTOR INITIAL: N

***** DISTRIBUTION OF THIS MATERIAL IS AS FOLLOWS *****

NOTES:

1. SEND 3 COPIES OF ALL MATERIAL TO I&E
ED REEVES-1 CY ALL MATERIAL

INCIDENT REPORTS
(DISTRIBUTION CODE A002)

FOR ACTION: BR CHIEF LWR#2 BC**W/4 ENCL

INTERNAL:

REG FILE**W/ENCL

I & E**W/3 ENCL

I & C SYSTEMS BR**W/ENCL

NOVAK/CHECK**W/ENCL

AD FOR ENG**W/ENCL

HANAUER**W/ENCL

AD FOR SYS & PROJ**W/ENCL

ENGINEERING BR**W/ENCL

KREGER/J. COLLINS**W/ENCL

K SEYFRIT/IE**W/ENCL

NRC PDR**W/ENCL

MIPC**W/3 ENCL

EMERGENCY PLAN BR**W/ENCL

EEB**W/ENCL

PLANT SYSTEMS BR**W/ENCL

AD FOR PLANT SYSTEMS**W/ENCL

REACTOR SAFETY BR**W/ENCL

VOLLMER/BUNCH**W/ENCL

POWER SYS BR**W/ENCL

EXTERNAL:

LPDR'S

ST. JOSEPH, MI**W/ENCL

TIC**W/ENCL

NSIC**W/ENCL

ACRS CAT B**W/16 ENCL

Ab/Occ. 24

DISTRIBUTION: LTR 45 ENCL 45
SIZE: 1P+1P+1P

CONTROL NBR: 781770010

***** THE END *****

~~NRC-3051305-011~~

UNITED STATES NUCLEAR REGULATORY
PDR SPECIAL WEEKLY LIS

~~06/01/78 - 06/09/78~~

FILE LEVELS

DOC. DATE

STN-50-568 NEW ENGLAND POWER COMPANY NEP #1

50

06/07/78

ACCESSION NBR: 77341-0153

TASK NB

568

DOCUMENT TYPE: MEMO

FICHE N

C

DOCUMENT SIZE: 1P

NOTARIZ

DOCKET DATE:

LPDR:

REPORT NBR:

RECP: VASSALLO D

RECP

ORG: TEDESCO R

ORG A

SUBJECT: REV. 1 TO THE DRAFT SAFETY EV



INDIANA & MICHIGAN POWER COMPANY

DONALD C. COOK NUCLEAR PLANT
P.O. Box 458, Bridgman, Michigan 49106

REGULATORY DOCKET FILE COPY

US NRC
DISTRIBUTION SERVICES
BRANCH

JUN 23 PM 12 10

RECEIVED DISTRIBUTION
SERVICES UNIT

June 19, 1978

Mr. J.G. Keppler, Regional Director
Office of Inspection and Enforcement
United States Nuclear Regulatory Commission
Region III
799 Roosevelt Road
Glen Ellyn, IL 60137

Operating License DPR-74
Docket No. 50-316

Dear Mr. Keppler:

Pursuant to the requirements of the Appendix A Technical Specifications
the following report is submitted:

RO 78-034/03L-0.

Sincerely,

D.V. Shaller
Plant Manager

/bab

cc: J.E. Dolan
R.W. Jurgensen
R.F. Kroeger
R. Kilburn
R.J. Vollen BPI
K.R. Baker RO:III
R.C. Callen MPSC
P.W. Steketee, Esq.
G. Charnoff, Esq.
R. Walsh, Esq.
G. Olson
J.M. Hennigan
PNSRC
J.F. Stietzel
Dir., IE (30 copies)
Dir., MIPC (3 copies)

781770010

A002
5/11

LICENSEE EVENT REPORT

CONTROL BLOCK:

1	2	3	4	5	6
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 (1)

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0	1
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 M I D C C 2

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4	5
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7 8 9 14 15 25 26 30 57 CAT 58CON'T

0	1
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 REPORT SOURCE L L

6	0	5	0	0	0	3	1	6
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7	0	5	1	9	7	8
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8	0	6	1	9	7	8
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9

7 8 60 61 DOCKET NUMBER 68 69 EVENT DATE 74 75 REPORT DATE 80

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0	2
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 | WHILE IN MODE 1 OPERATION REACTOR COOLANT SYSTEM UNIDENTIFIED LEAKAGE EX

0	3
---	---

 | CEDED TECH. SPEC. 3.4.6.2.b LIMIT. UNIT WAS STARTED DOWN 4½ HOURS AFTER

0	4
---	---

 | DISCOVERY BUT WAS RETURNED TO ORIGINAL LOADING WHEN LEAKAGE INDICATED WI

0	5
---	---

 | THIN LIMITS. LEAKAGE EXCEEDED LIMITS THE SECOND TIME. THE UNIT WAS STARTED

0	6
---	---

 | DOWN 4 HOURS AFTER DISCOVERY AND REMOVED FROM SERVICE 2½ HOURS LATER. THE

0	7
---	---

 | REACTOR WAS COOLED DOWN TO MODE 5 IN THE NEXT 17 HOURS.

0	8
---	---

 |
7 8 9 80

0	9
---	---

 SYSTEM CODE C E (11) CAUSE CODE E (12) CAUSE SUBCODE X (13) COMPONENT CODE V A L V E X (14) COMP. SUBCODE E (15) VALVE SUBCODE D (16)
7 8 9 10 11 12 13 18 19 20
(17) LER/RO REPORT NUMBER 7 8 EVENT YEAR 7 8 SEQUENTIAL REPORT NO. 0 3 4 OCCURRENCE CODE 0 3 REPORT TYPE 1 REVISION NO. 0
21 22 23 24 26 27 28 29 30 31 32
ACTION TAKEN FUTURE ACTION EFFECT ON PLANT SHUTDOWN METHOD HOURS ATTACHMENT SUBMITTED NPRD-4 FORM SUB. PRIME COMP. SUPPLIER COMPONENT MANUFACTURER
A (18) Z (19) A (20) A (21) 0 2 8 7 Y (23) Y (24) N (25) A 5 A 5 (26)
33 34 35 36 37 40 41 42 43 44 47

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1	0
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 | ISOLATION VALVE BETWEEN RCS AND RHR FOUND TO HAVE EXCESSIVE STEM LEAKOFF

1	1
---	---

 | AND PACKING LEAKAGE. VALVE REPACKED AND STEM LEAKOFF ELIMINATED. RCS RETUR

1	2
---	---

 | NED TO OPERATING PRESS. AND TEMP. AND LEAKAGE WAS STILL EXCESSIVE. SAFETY

1	3
---	---

 | VALVE ON FAILED FUEL DETECTOR FOUND LEAKING THROUGH. SAFETY VALVE REPLAC

1	4
---	---

 | ED AND LEAKAGE WAS REDUCED TO NORMAL
7 8 9 80FACILITY STATUS E (28) % POWER 0 8 0 (29) NA OTHER STATUS (30) METHOD OF DISCOVERY B (31) DISCOVERY DESCRIPTION ROUTINE SURVEILLANCE (32)
7 8 9 10 12 13 44 45 46 80
ACTIVITY CONTENT RELEASED OF RELEASE Z (33) Z (34) NA AMOUNT OF ACTIVITY (35) LOCATION OF RELEASE (36)
7 8 9 10 11 44 45 80PERSONNEL EXPOSURES NUMBER 0 0 0 (37) TYPE - (38) NA DESCRIPTION (39)
7 8 9 11 12 13 80PERSONNEL INJURIES NUMBER 0 0 0 (40) DESCRIPTION (41)
7 8 9 11 12 80LOSS OF OR DAMAGE TO FACILITY TYPE Z (42) NA DESCRIPTION (43)
7 8 9 10 80PUBLCITY ISSUED N (44) NA DESCRIPTION (45)
7 8 9 10 80

NRC USE ONLY

NAME OF PREPARER R. S. Keith

PHONE: (616) 465-5901

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS

INITIAL INSPECTION FOUND EXCESSIVE LEAKAGE FROM THE PACKING OF THE ISOLATION VALVE BETWEEN THE REACTOR COOLANT SYSTEM AND THE RESIDUAL HEAT REMOVAL SYSTEM. THE PACKING BOX OF THIS VALVE WAS ALSO EQUIPPED WITH A STEAM LEAKOFF PIPED TO THE REACTOR COOLANT DRAIN TANK. THIS VALVE IS A 14" TWIN DISK GATE MANUFACTURED BY COPES VULCAN WITH A 2500 PSIG RATING. THE VALVE WAS REPACKED AND THE STEAM LEAKOFF WAS DISCONNECTED AND TERMINATED IN BOTH DIRECTIONS.

WHEN THE REACTOR COOLANT SYSTEM WAS RETURNED TO OPERATING PRESSURE AND TEMPERATURE THE LEAKAGE RATE WAS STILL IN EXCESS OF ALLOWABLE LIMITS.

A 75 PSIG SET SAFETY VALVE IN THE FAILED FUEL DETECTOR SYSTEM WAS FOUND TO BE LEAKING THROUGH. THIS FINDING WAS 4 3/4 HOURS AFTER THE LEAK RATE BECAME KNOWN. THE SAFETY VALVE WAS ISOLATED AND THE LEAKAGE RATE REDUCED TO NORMAL.

THIS SAFETY VALVE IS A 1/2" INLINE MANUFACTURED BY NUPROCO. MODEL # 4A SS 8 CPA2-50. IT WAS REPLACED BY A DUPLICATE. THE FAILED VALVE WAS NOT DISASSEMBLED TO DETERMINE FAILURE DUE TO HIGH CONTAMINATION.

D. Sanham
FILE



INDIANA & MICHIGAN POWER COMPANY

DONALD C. COOK NUCLEAR PLANT
P.O. Box 458, Bridgman, Michigan 49106

June 19, 1978

Mr. J.G. Keppler, Regional Director
Office of Inspection and Enforcement
United States Nuclear Regulatory Commission,
Region III
799 Roosevelt Road
Glen Ellyn, IL 60137

Operating License DPR-74
Docket No. 50-316

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Sincerely,

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cc: J.E. Dolan
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Dir., IE (30 copies)
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A/O 4

JUN 21 1978

GP

LICENSEE EVENT REPORT

CONTROL BLOCK:

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0	1	M	I	D	C	C	2	2	0	0	0	0	0	0	0	0	0	0	0	3	4	1	1	1	1	4			5			
7	8	LICENSEE CODE						14	15	LICENSE NUMBER											25	26	LICENSE TYPE					30	57	CAT		58

CON'T

REPORT SOURCE 01 60 61 0 5 0 0 0 3 1 6 7 0 5 1 9 7 8 8 0 6 1 9 7 8 9

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0 2 | WHILE IN MODE 1 OPERATION REACTOR COOLANT SYSTEM UNIDENTIFIED LEAKAGE EX
0 3 | CEDED TECH. SPEC. 3.4.6.2.b LIMIT. UNIT WAS STARTED DOWN 4½ HOURS AFTER
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0 5 | THIN LIMITS. LEAKAGE EXCEEDED LIMITS THE SECOND TIME. THE UNIT WAS STARTED
0 6 | DOWN 4 HOURS AFTER DISCOVERY AND REMOVED FROM SERVICE 2½ HOURS LATER. THE
0 7 | REACTOR WAS COOLED DOWN TO MODE 5 IN THE NEXT 17 HOURS.
0 8 |

0 9		SYSTEM CODE		CAUSE CODE		CAUSE SUBCODE		COMPONENT CODE				COMP. SUBCODE		VALVE SUBCODE								
0	9	C	E	11	E	12	X	13	V	A	L	V	E	X	14	E	15	D	16			
7	8	9	10	11	12	13	14	15	16	17	18	19	20	21	22	23	24	25	26			
LER/RO REPORT NUMBER		EVENT YEAR		SEQUENTIAL REPORT NO.		OCCURRENCE CODE		REPORT TYPE		REVISION NO.												
17	7	8	21	22	23	0	3	4	27	0	3	30	31	32	0	33	34	35	36			
ACTION TAKEN		FUTURE ACTION		EFFECT ON PLANT		SHUTDOWN METHOD		HOURS		ATTACHMENT SUBMITTED		NPRD-4 FORM SUB.		PRIME COMP. SUPPLIER		COMPONENT MANUFACTURER						
A	18	Z	19	A	20	A	21	0	2	8	7	Y	23	Y	24	N	25	A	5	A	5	26
33	34	35	36	37	38	39	40	41	42	43	44	45	46	47	48	49	50	51	52	53	54	55

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 0 ISOLATION VALVE BETWEEN RCS AND RHR FOUND TO HAVE EXCESSIVE STEM LEAKOFF

1 1 AND PACKING LEAKAGE. VALVE REPACKED AND STEM LEAKOFF ELIMINATED. RCS RETUR

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1 3 VALVE ON FAILED FUEL DETECTOR FOUND LEAKING THROUGH. SAFETY VALVE REPLACE

1 4 D AND LEAKAGE WAS REDUCED TO NORMAL

FACILITY STATUS		% POWER		OTHER STATUS		METHOD OF DISCOVERY		DISCOVERY DESCRIPTION	
1	5	E	28	0	8	0	29	NA	30
ACTIVITY RELEASED		CONTENT OF RELEASE		AMOUNT OF ACTIVITY		LOCATION OF RELEASE		31	
1	6	Z	33	Z	34	NA	35	36	37
PERSONNEL EXPOSURES		NUMBER		TYPE		DESCRIPTION		38	
1	7	0	0	0	37		38	NA	39
PERSONNEL INJURIES		NUMBER		DESCRIPTION		40		41	
1	8	0	0	0	40	NA	41	42	
LOSS OF OR DAMAGE TO FACILITY		TYPE		DESCRIPTION		43		44	
1	9	Z	42	NA	43	44		45	
PUBLICITY ISSUED		DESCRIPTION		45		46		47	
2	0	N	44	NA	45	46		47	

NAME OF PREPARER R. S. Keith

PHONE: (616) 465-5901

RECEIVED THE OFFICE OF THE ATTORNEY GENERAL
JAN 10 1964

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