

A 06/26/78

REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)
DISTRIBUTION FOR INCOMING MATERIAL

50-316

REC: KEPPLER J G
NRC

ORG: SHALLER D V
IN & MI PWR

DOCDATE: 06/21/78
DATE RCVD: 06/23/78

DOCTYPE: LETTER NOTARIZED: NO

COPIES RECEIVED

SUBJECT:

LTR 1 ENCL 1

FORWARDING LICENSEE EVENT REPT (RO 50-316/78-035) ON 05/19/78 CONCERNING
WHILE IN MODE 1 OPERATION REACTOR COOLANT SYSTEM UNIDENTIFIED LEAKAGE
EXCEEDED TECH SPEC 3. 4. 6. 2. B... W/ATT.

PLANT NAME: COOK - UNIT 2

REVIEWER INITIAL: XJM

DISTRIBUTOR INITIAL: *ne*

***** DISTRIBUTION OF THIS MATERIAL IS AS FOLLOWS *****

NOTES:

1. SEND 3 COPIES OF ALL MATERIAL TO I&E
ED REEVES-1 CY ALL MATERIAL

INCIDENT REPORTS
(DISTRIBUTION CODE A002)

FOR ACTION: ~~BR~~ CHIEF LWR#2 BC**W/4 ENCL

INTERNAL:

REG FILE**W/ENCL

I & E**W/3 ENCL

I & C SYSTEMS BR**W/ENCL

NOVAK/CHECK**W/ENCL

AD FOR ENG**W/ENCL

HANAUER**W/ENCL

AD FOR SYS & PROJ**W/ENCL

ENGINEERING BR**W/ENCL

KREGER/J. COLLINS**W/ENCL

K SEYFRIT/IE**W/ENCL

NRC PDR**W/ENCL

MIPC**W/3 ENCL

EMERGENCY PLAN BR**W/ENCL

EEB**W/ENCL

PLANT SYSTEMS BR**W/ENCL

AD FOR PLANT SYSTEMS**W/ENCL

REACTOR SAFETY BR**W/ENCL

VOLLMER/BUNCH**W/ENCL

POWER SYS BR**W/ENCL

EXTERNAL:

LPDR'S

ST. JOSEPH, MI**W/ENCL

TIC**W/ENCL

NSIC**W/ENCL

ACRS CAT B**W/16 ENCL

Do/acc f

DISTRIBUTION: LTR 45 ENCL 45
SIZE: 1P+1P+1P

CONTROL NBR: 781770011

***** THE END ***** *BD*

NRC-3031505-011

UNITED STATES NUCLEAR REGULATORY
PDR SPECIAL WEEKLY LI

06/01/78 - 06/09/78

FILE LEVELS

DOC. DATE

STN-50-568 NEW ENGLAND POWER COMPANY NEP #1

50

06/01/78

ACCESSION NBR: 78152-0037

TASK NE

568

DOCUMENT TYPE: MEMO

FICHE M

C

DOCUMENT SIZE: 1P

NOTARIZ

DOCKET DATE:

LPDR:

REPORT NBR:

RECP: BOYD R S

RECP

ORG: PARR O D

ORG /

SUBJECT: STATUS OF THE SAFETY EVALUATI

50

06/01/78

ACCESSION NBR: 78152-0036

TASK NE

568

DOCUMENT TYPE: LETTER

FICHE M

C

DOCUMENT SIZE: 1P

NOTARIZ

DOCKET DATE:

LPDR:

REPORT NBR:

RECP: LAWROSKI S

RECP

ORG: VASSALLO D

ORG A

SUBJECT: FORWARDING SAFETY EVALUATION

W/O ENCL.

50

06/06/78

ACCESSION NBR: 77341-0166

TASK NB

568

DOCUMENT TYPE: MEMO

FICHE M

C

DOCUMENT SIZE: 1P

NOTARIZ

DOCKET DATE:

LPDR:

REPORT NBR:

RECP: VASSALLO D B

RECP

ORG: NORIAN P E

ORG A

SUBJECT: SUPPLEMENTARY QUESTIONS - NEW



INDIANA & MICHIGAN POWER COMPANY

DONALD C. COOK NUCLEAR PLANT
P.O. Box 458, Bridgman, Michigan 49106

REGULATORY DOCKET FILE COPY

RECEIVED DISTRIBUTION
SERVICES UNIT

1978 JUN 23 AM 9 48

US NRC
DISTRIBUTION SERVICES
BRANCH

June 21, 1978

Mr. J.G. Keppler, Regional Director
Office of Inspection and Enforcement
United States Nuclear Regulatory Commission
Region III
799 Roosevelt Road
Glen Ellyn, IL 60137

Operating License DPR-74
Docket No. 50-316

Dear Mr. Keppler:

Pursuant to the requirements of the Appendix A Technical Specifications
the following report is submitted:

RO 78-035/03L-0.

This report is a resubmittal of our License Event Report No. 78-034/03L-0
dated June 19, 1978. A Licensee Event Report by the same number was
previously submitted on June 13, 1978.

Sincerely,

D.V. Shaller
Plant Manager

/bab

cc: J.E. Dolan
R.W. Jurgensen
R.F. Kroeger
R. Kilburn
R.J. Vollen BPI
K.R. Baker RO:III
R.C. Callen MPSC
P.W. Steketee, Esq.
R. Walsh, Esq.
G. Charnoff, Esq.
G. Olson
J.M. Hennigan
PNSRC
J.F. Stietzel
Dir., IE (30 copies)
Dir., MIPC (3 copies)

781770011

A002
3/11

CORRECTED COPY

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

CON'T

0	1
7	8

REPORT SOURCE

L	6	0	5	0	0	0	3	1	6	7	0	5	1	9	7	8	8	0	6	1	9	7	8	9
60	61									68	69						74	75						80
DOCKET NUMBER										EVENT DATE										REPORT DATE				

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0 8 | _____ 8

09		SYSTEM CODE C E		CAUSE CODE E	CAUSE SUBCODE X	COMPONENT CODE V A L V E X		COMP. SUBCODE E	VALVE SUBCODE D
7	8	9	10	11	12	13	14	15	16
(17) LER/RO REPORT NUMBER		EVENT YEAR 7 8			SEQUENTIAL REPORT NO. 0 3 5		OCCURRENCE CODE 0 3	REPORT TYPE L	REVISION NO. 0
21		22		23	24		26	27	28
ACTION TAKEN A		FUTURE ACTION Z		EFFECT ON PLANT A	SHUTDOWN METHOD A		HOURS 0 2 8 7	ATTACHMENT SUBMITTED Y	NPRD-4 FORM SUB. Y
33	34	35	36	37	38	39	40	41	42
(18)	(19)	(20)	(21)	(22)	(23)	(24)	(25)	(26)	(27)
PRIME COMP. SUPPLIER N		COMPONENT MANUFACTURER A 5 A 5							
43		44		45		46		47	

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 0 ISOLATION VALVE BETWEEN RCS AND RHR FOUND TO HAVE EXCESSIVE STEM LEAKOFF

1 1 AND PACKAGE LEAKAGE. VALVE REPACKED AND STEM LEAKOFF ELIMINATED. RCS RET

1 2 URNED TO OPERATING PRESS. AND TEMP. AND LEAKAGE WAS STILL EXCESSIVE. SAF

1 3 ETY VALVE ON FAILED FUEL DETECTOR FOUND LEAKING THROUGH. SAFETY VALVE RE

1 4 PLACED AND LEAKAGE WAS REDUCED TO NORMAL.

7 8 9
FACILITY STATUS % POWER OTHER STATUS (30) METHOD OF DISCOVERY DISCOVERY DESCRIPTION (32)
1 5 E (28) 0 8 0 (29) NA B (31) ROUTINE SURVEILLANCE

7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60

ACTIVITY CONTENT
RELEASED OF RELEASE AMOUNT OF ACTIVITY (35) LOCATION OF RELEASE (36)

1 6 Z 33 Z 34 NA NA

2 3 4 5 6 7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60

PERSONNEL EXPOSURES									
NUMBER			TYPE	DESCRIPTION					
1	7	0	0	0					
			37	Z					
			38	NA					

7		8		9		11		12		13		
PERSONNEL INJURIES												
NUMBER						DESCRIPTION (41)						
1	8	0	0	0	40	NA						

7		8	9	11	12
		LOSS OF OR DAMAGE TO FACILITY (43)			
		TYPE		DESCRIPTION	
1	9	Z	(42)	NA	

7 8 9 10
PUBLICITY
ISSUED (44) DESCRIPTION (45)
2 0 N NA
68 69 70 71 72 73 74 75 76 77 78 79 80
NRC USE ONLY

PHONE: (616) 465-5901

SUPPLEMENT TO LER No. 78-035/03L-0

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS

INITIAL INSPECTION FOUND EXCESSIVE LEAKAGE FROM THE PACKING OF THE ISOLATION VALVE BETWEEN THE REACTOR COOLANT SYSTEM AND THE RESIDUAL HEAT REMOVAL SYSTEM. THE PACKING BOX OF THIS VALVE WAS ALSO EQUIPPED WITH A STEAM LEAKOFF PIPED TO THE REACTOR COOLANT DRAIN TANK. THIS VALVE IS A 14" TWIN DISK GATE MANUFACTURED BY COPEL VULCAN WITH A 2500 PSIG RATING. THE VALVE WAS REPACKED AND THE STEAM LEAKOFF WAS DISCONNECTED AND TERMINATED IN BOTH DIRECTIONS.

WHEN THE REACTOR COOLANT SYSTEM WAS RETURNED TO OPERATING PRESSURE AND TEMPERATURE THE LEAKAGE RATE WAS STILL IN EXCESS OF ALLOWABLE LIMITS.

A 75 PSIG SET SAFETY VALVE IN THE FAILED FUEL DETECTOR SYSTEM WAS FOUND TO BE LEAKING THROUGH. THIS FINDING WAS 4 3/4 HOURS AFTER THE LEAK RATE BECAME KNOWN. THE SAFETY VALVE WAS ISOLATED AND THE LEAKAGE RATE REDUCED TO NORMAL.

THIS SAFETY VALVE IS A 1/2" INLINE MANUFACTURED BY NUPROCO. MODEL #4A SS 8 CPA2-50. IT WAS REPLACED BY A DUPLICATE. THE FAILED VALVE WAS NOT DISASSEMBLED TO DETERMINE FAILURE DUE TO HIGH CONTAMINATION.

D. Sarham
FILE



INDIANA & MICHIGAN POWER COMPANY

DONALD C. COOK NUCLEAR PLANT
P.O. Box 458, Bridgman, Michigan 49106

June 21, 1978

Mr. J.G. Keppler, Regional Director
Office of Inspection and Enforcement
United States Nuclear Regulatory Commission
Region III
799 Roosevelt Road
Glen Ellyn, IL 60137

Operating License DPR-74
Docket::No. 50-316

Dear Mr. Keppler:

Pursuant to the requirements of the Appendix A Technical Specifications
the following report is submitted:

RO 78-035/03L-0.

This report is a resubmittal of our License Event Report No. 78-034/03L-0
dated June 19, 1978. A Licensee Event Report by the same number was
previously submitted on June 13, 1978.

Sincerely,

D.V. Shaller
Plant Manager

/bab

cc: J.E. Dolan
R.W. Jurgensen
R.F. Kroeger
R. Kilburn
R.J. Vollen BPI
K.R. Baker RO:III
R.C. Callen MPSC
P.W. Steketee, Esq.
R. Walsh, Esq.
G. Charnoff, Esq.
G. Olson
J.M. Hennigan
PNSRC
J.F. Stietzel
Dir., IE (30 copies)
Dir., MIPC (3 copies)

JUN 23 1978

A/04

GO

1950-1951

SUPPLEMENT TO LER No. 78-035/03L-0

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS

INITIAL INSPECTION FOUND EXCESSIVE LEAKAGE FROM THE PACKING OF THE ISOLATION VALVE BETWEEN THE REACTOR COOLANT SYSTEM AND THE RESIDUAL HEAT REMOVAL SYSTEM. THE PACKING BOX OF THIS VALVE WAS ALSO EQUIPPED WITH A STEAM LEAKOFF PIPED TO THE REACTOR COOLANT DRAIN TANK. THIS VALVE IS A 14" TWIN DISK GATE MANUFACTURED BY COPE'S VULCAN WITH A 2500 PSIG RATING. THE VALVE WAS REPACKED AND THE STEAM LEAKOFF WAS DISCONNECTED AND TERMINATED IN BOTH DIRECTIONS.

WHEN THE REACTOR COOLANT SYSTEM WAS RETURNED TO OPERATING PRESSURE AND TEMPERATURE THE LEAKAGE RATE WAS STILL IN EXCESS OF ALLOWABLE LIMITS.

A 75 PSIG SET SAFETY VALVE IN THE FAILED FUEL DETECTOR SYSTEM WAS FOUND TO BE LEAKING THROUGH. THIS FINDING WAS 4 3/4 HOURS AFTER THE LEAK RATE BECAME KNOWN. THE SAFETY VALVE WAS ISOLATED AND THE LEAKAGE RATE REDUCED TO NORMAL.

THIS SAFETY VALVE IS A 1/2" INLINE MANUFACTURED BY NUPROCO. MODEL #4A SS 8 CPA2-50. IT WAS REPLACED BY A DUPLICATE. THE FAILED VALVE WAS NOT DISASSEMBLED TO DETERMINE FAILURE DUE TO HIGH CONTAMINATION.

