



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

July 31, 2018

MEMORANDUM TO: Samuel S. Lee, Chief
Licensing Branch 1
Division of Licensing, Siting,
and Environmental Analysis
Office of New Reactors

FROM: Omid Tabatabai, Senior Project Manager /RA/
Licensing Branch 1
Division of Licensing, Siting,
and Environmental Analysis
Office of New Reactors

SUBJECT: SUMMARY OF THE JULY 11, 2018, PUBLIC MEETING WITH
NUSCALE POWER, LLC, TO DISCUSS CHANGES TO TABLE
17.4-1 IN THE NUSCALE DESIGN CERTIFICATION
APPLICATION, REVISION 1, TIER 2, CHAPTER 17, "QUALITY
ASSURANCE AND RELIABILITY ASSURANCE"

On July 11, 2018, representatives of the U.S. Nuclear Regulatory Commission (NRC) and NuScale Power, LLC (NuScale) held a public teleconference meeting to discuss the NRC staff's questions related to the NuScale Design Certification Application, Tier 2, Chapter 17, "Quality Assurance and Reliability Assurance."

Enclosure 1 captures the summary of the topics discussed during the teleconference. The agenda and list of meeting attendees are included in Enclosures 2 and 3, respectively. The meeting notice for this meeting is available in the Agencywide Documents Access and Management System under Accession No. ML18170A243.

Docket No. 52-048

Enclosures:

1. Meeting Summary
2. Agenda
3. Attendees

CONTACT: Omid Tabatabai, NRO/DLSE
301-415-6616

SUBJECT: SUMMARY OF THE JULY 11, 2018, PUBLIC MEETING WITH NUSCALE POWER, LLC, TO DISCUSS CHANGES TO TABLE 17.4-1 IN THE NUSCALE DESIGN CERTIFICATION APPLICATION (DCA), REVISION 1, TIER 2, CHAPTER 17, "QUALITY ASSURANCE AND RELIABILITY ASSURANCE"
DATED: JULY 31, 2018

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DATE	7/23/2018	7/30/2018	7/31/2018	7/31/2018

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U.S. NUCLEAR REGULATORY COMMISSION

SUMMARY OF THE JULY 11, 2018, TELECONFERENCE WITH NUSCALE POWER, LLC

TO DISCUSS THE STAFF'S QUESTIONS RELATED TO CHANGES TO TABLE 17.4-1 IN

CHAPTER 17, "QUALITY ASSURANCE AND RELIABILITY ASSURANCE"

The purpose of this meeting was for NuScale Power, LLC (NuScale) to provide an explanation for removing the chemical and volume control system (CVCS) components from Table 17.4-1 in Revision 1 of the final safety analysis report (FSAR).

Nuscale provided the U.S. Nuclear Regulatory Commission (NRC) staff with a background of why the valves were included in Revision 0, what changed and what led to its removal from the design reliability assurance program (D-RAP) systems, structures and components (SSC) table in Revision 1 of Chapter 17.4. Nuscale stated that the valves are used to stop the supply of demineralized water to the CVCS system, which otherwise could cause a plant transient. Nuscale said the failure of the valves to close is modeled in the probabilist risk assessment (PRA) as part of the general transient initiating event. The valves were included in Revision 0 of Table 17.4-1 because at the time of issuing Revision 0, the PRA was not finalized, therefore, the expert panel conservatively added the valves and their function to the RAP list based on the potential to cause a plant transient. When the PRA was finalized, the valves and the general transient were not identified as risk significant. The expert panel subsequently removed the valves from Table 17.4-1 as reflected in Revision 1 of the FSAR. The NRC staff asked clarifying questions about the valve function, system interface, and how it's modeled in the PRA. The NRC staff informed Nuscale that the information will be discussed internally to determine the next steps.

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SUMMARY OF THE JULY 11, 2018, TELECONFERENCE
WITH NUSCALE POWER, LLC

MEETING AGENDA

Introduction	1:00 – 1:10 pm
Staff's questions related to Table 17.4-1	1:10 – 1:45 pm
Public Comments	1:55 – 2:00 pm

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SUMMARY OF THE JULY 11, 2018, TELECONFERENCE
WITH NUSCALE POWER, LLC

LIST OF ATTENDEES

NuScale Power, LLC

David Holick
Sarah Bristol
Cindy Williams
Paul Infanger
Russell Goff
Patrick Conley

U.S. Nuclear Regulatory Commission Staff

Michelle Hayes
Odunayo Ayegbusi
Omid Tabatabai