

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)	DOCKET NUMBER (2)	PAGE (3)	
		1 OF 03	
Surry Power Station, Units 1 & 2	05000280		

TITLE (4)	Required Surveillance Not Performed Due To Human Error
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EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)										
MONTH	DAY	YEAR	YEAR		SEQUENTIAL NUMBER		REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES					DOCKET NUMBER(S)				
0	7	17	8	7	0	3	5	0	0	1	2	3	1	8	7	0 5 0 0 0				

OPERATING MODE (8)		N	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)									
POWER LEVEL (10)	1 0 0		20.402(b)		20.405(c)	X	50.73(a)(2)(iv)		73.71(b)			
		20.405(a)(1)(i)		50.38(c)(1)		50.73(a)(2)(v)		73.71(c)				
		20.405(a)(1)(ii)		50.38(c)(2)		50.73(a)(2)(vii)		OTHER (Specify in Abstract below and in Text, NRC Form 366A)				
		20.405(a)(1)(iii)		50.73(a)(2)(i)		50.73(a)(2)(viii)(A)						
		20.405(a)(1)(iv)		50.73(a)(2)(ii)		50.73(a)(2)(viii)(B)						
20.405(a)(1)(v)		50.73(a)(2)(iii)		50.73(a)(2)(x)								

LICENSEE CONTACT FOR THIS LER (12)									
NAME						TELEPHONE NUMBER			
D. L. Benson, Station Manager						AREA CODE			
						8 0 4		3 5 7 - 3 1 8 4	

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)											
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	

SUPPLEMENTAL REPORT EXPECTED (14)		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE)	<input checked="" type="checkbox"/> NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On July 17, 1987, with Units 1 and 2 at 100% power, it was determined that the Emergency Diesel Generator (EDG) {EIIS-DG} degraded/undervoltage protection {EIIS-EB} surveillance test was not being performed. This test demonstrates that the degraded/undervoltage protection is defeated when the EDG is the sole source of power to an emergency bus, and that the protection is reinstated when the diesel output breaker is open. Technical Specification 4.6.1.b requires that this test be performed during each refueling shutdown. Failure to perform the diesel degraded/undervoltage surveillance as required by Technical Specifications was caused by a failure to incorporate the requirements of a Technical Specification amendment into the existing test program. This was determined to be a reportable occurrence by the Station Nuclear Safety and Operating Committee on December 4, 1987. Special Tests for functional testing of the enable/disable interlocks were performed and completed on the emergency buses in September, 1987. This testing verified that the relays are fully operable and that the protection matrix functions as designed. The refueling Periodic Tests will be revised to include testing of the enable/disable interlock. The revisions will be completed prior to the next refueling outage.

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PDR ADOCK 05000280
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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104

EXPIRES: 8/31/88

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
Surry Power Station, Units 1 & 2	0500028087	03	5	0	02	OF 03

TEXT (If more space is required, use additional NRC Form 365A's) (17)

1.0 Description of the Event

On July 17, 1987, with Units 1 and 2 at 100% power, it was determined that the Emergency Diesel Generator (EDG) {EIIS-DG} degraded/undervoltage protection {EIIS-EB} surveillance test was not being performed. This test demonstrates that the degraded/undervoltage protection is defeated when the EDG is the sole source of power to an emergency bus, and that the protection is reinstated when the diesel output breaker is open. Technical Specification 4.6.1.b requires that this test be performed during each refueling shutdown. This was determined to be a reportable occurrence by the Station Nuclear Safety and Operating Committee on December 4, 1987.

2.0 Safety Consequences and Implications

The tests specified in Technical Specification Section 4.6 are designed to demonstrate that the EDGs will provide power for operation of essential safeguards equipment. They also assure that the EDG system controls and the control systems for the safeguards equipment will function automatically in the event of a loss of normal station service power. Technical reviews have been performed to determine the effect of failure of an untested circuit. It was determined that the impact on safety was minimal in that no single failure could cause a loss of the safety function. Therefore, the health and safety of the public were not affected.

3.0 Cause

Failure to perform the diesel degraded/undervoltage surveillance as required by Technical Specifications was caused by a failure to incorporate the requirements of a Technical Specification amendment into the existing test program.

4.0 Immediate Corrective Action(s)

Special Tests (1-ST-206, 2-ST-206) for functional testing of the enable/disable interlocks were performed and completed on the emergency buses in September, 1987. This testing verified that the relays are fully operable and that the protection matrix functions as designed.

NRC Form 366A
(9-83)

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

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APPROVED OMB NO. 3150-0104

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TEXT (If more space is required, use additional NRC Form 366A's) (17)

5.0 Additional Corrective Action(s)

None required.

6.0 Actions Taken to Prevent Recurrence

The refueling Periodic Tests (1, 2-PT-18.2A, B) will be revised to include testing of the enable/disable interlock. The revisions will be completed prior to the next refueling outage.

7.0 Similar Events

None.

8.0 Manufacturer/Model Number

N/A

VIRGINIA ELECTRIC AND POWER COMPANY
Surry Power Station
P. O. Box 315
Surry, Virginia 23883

December 31, 1987

U.S. Nuclear Regulatory Commission
Document Control Desk
016 Phillips Building
Washington, D.C. 20555

Serial No.: 87-043
Docket No.: 50-280
Docket No.: 50-281
Licensee No.: DPR-32
Licensee No.: DPR-37

Gentlemen:

Pursuant to Surry Power Station Technical Specifications, Virginia Electric and Power Company hereby submits the following Licensee Event Report for Surry Units 1 and 2.

REPORT NUMBER

87-035-00

This report has been reviewed by the Station Nuclear Safety and Operating Committee and will be reviewed by Safety Evaluation and Control.

Very truly yours,

David L. Benson

David L. Benson
Station Manager

Enclosure

cc: Dr. J. Nelson Grace
Regional Administrator
Suite 2900
101 Marietta Street, NW
Atlanta, Georgia 30323

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