

ATTACHMENT 1

PROPOSED TECHNICAL SPECIFICATION CHANGE

SURRY POWER STATION
UNIT NOS. 1 AND 2

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6. Any other matter involving safe operation of the nuclear power stations which is referred to the Director - Safety Evaluation and Control.

7. Reports and meeting minutes of the Station Nuclear Safety and Operating Committee.

f. Authority

The Supervisor-IOER shall report to and advise the Director - SEC regarding any safety concern discovered during the independent review process the Director - Safety Evaluation and Control shall report to and advise the Manager - Nuclear Programs and Licensing, who shall advise the Vice President - Nuclear Operations on those areas of responsibility specified in Section 6.1.C.2.e.

g. Records

Records of IOER activities required by Specification 6.1.C.2.e shall be prepared and maintained in the SEC files and a summary shall be disseminated each calendar month as follows:

1. Vice President - Nuclear Operations
2. Nuclear Power Station Managers
3. Manager - Nuclear Operations Support
4. Manager - Nuclear Programs and Licensing
5. Manager - Corporate Quality Assurance
6. Others that the Director - Safety Evaluation and Control may designate

6. The Station Security Plan and implementing procedures at least once per 12 months.
7. Any other area of facility operation considered appropriate by the Manager - Corporate Quality Assurance or the Senior Vice President - Power Operations.
8. The Station Fire Protection Program and implementing procedures at least once per 24 months.
9. An independent fire protection and loss prevention program inspection and audit shall be performed at least once per 12 months utilizing either qualified offsite licensee personnel or an outside fire protection firm.
10. An inspection and audit of the fire protection and loss prevention program shall be performed by a qualified outside fire consultant at least once per 36 months.
11. The radiological environmental monitoring program at least once per 12 months.
12. The Offsite Dose Calculation Manual and implementing procedures at least once per 24 months.

13. The Process Control Program and implementing procedures for processing and packaging of radioactive waste at least once per 24 months.

b. Authority

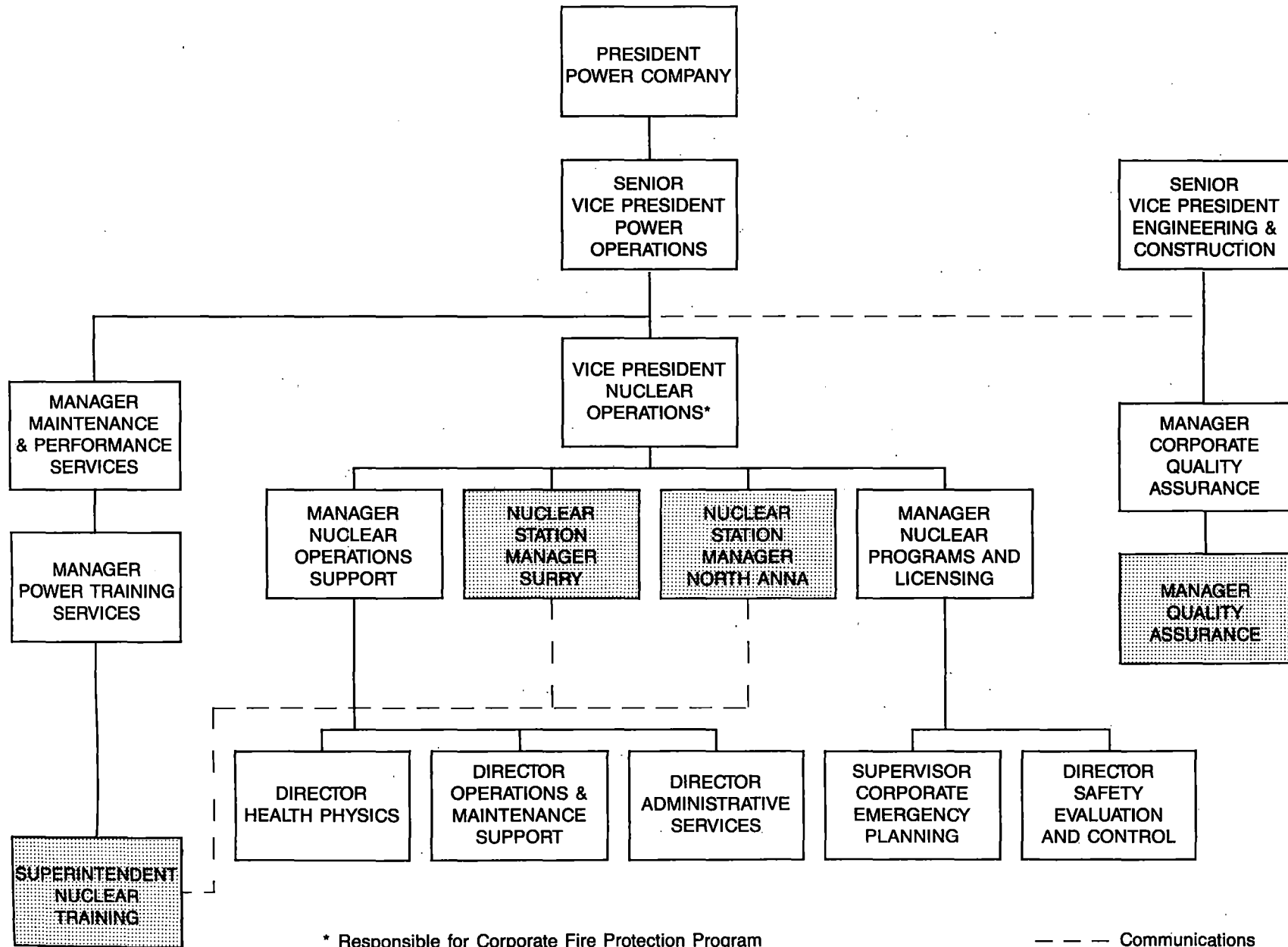
The Quality Assurance Department shall report to and advise the Manager - Corporate Quality Assurance, who shall advise the Senior Vice President - Engineering and Construction on those areas of responsibility specified in Specification 6.1.C.3.a above.

c. Records

Records of the Quality Assurance Department audits shall be prepared and maintained in the department files. Audit reports shall be disseminated as indicated below:

1. Vice President - Nuclear Operations
2. Nuclear Power Station Manager
3. Manager - Nuclear Operations Support
4. Manager - Corporate Quality Assurance
5. Manager - Nuclear Programs and Licensing
6. Director - Safety Evaluation and Control
7. Supervisor of area audited
8. Nuclear Power Station Manager - Quality Assurance

Figure 6.1-1 Offsite Organization for Facility Management and Technical Support



* Responsible for Corporate Fire Protection Program

--- Communications

▒ Onsite

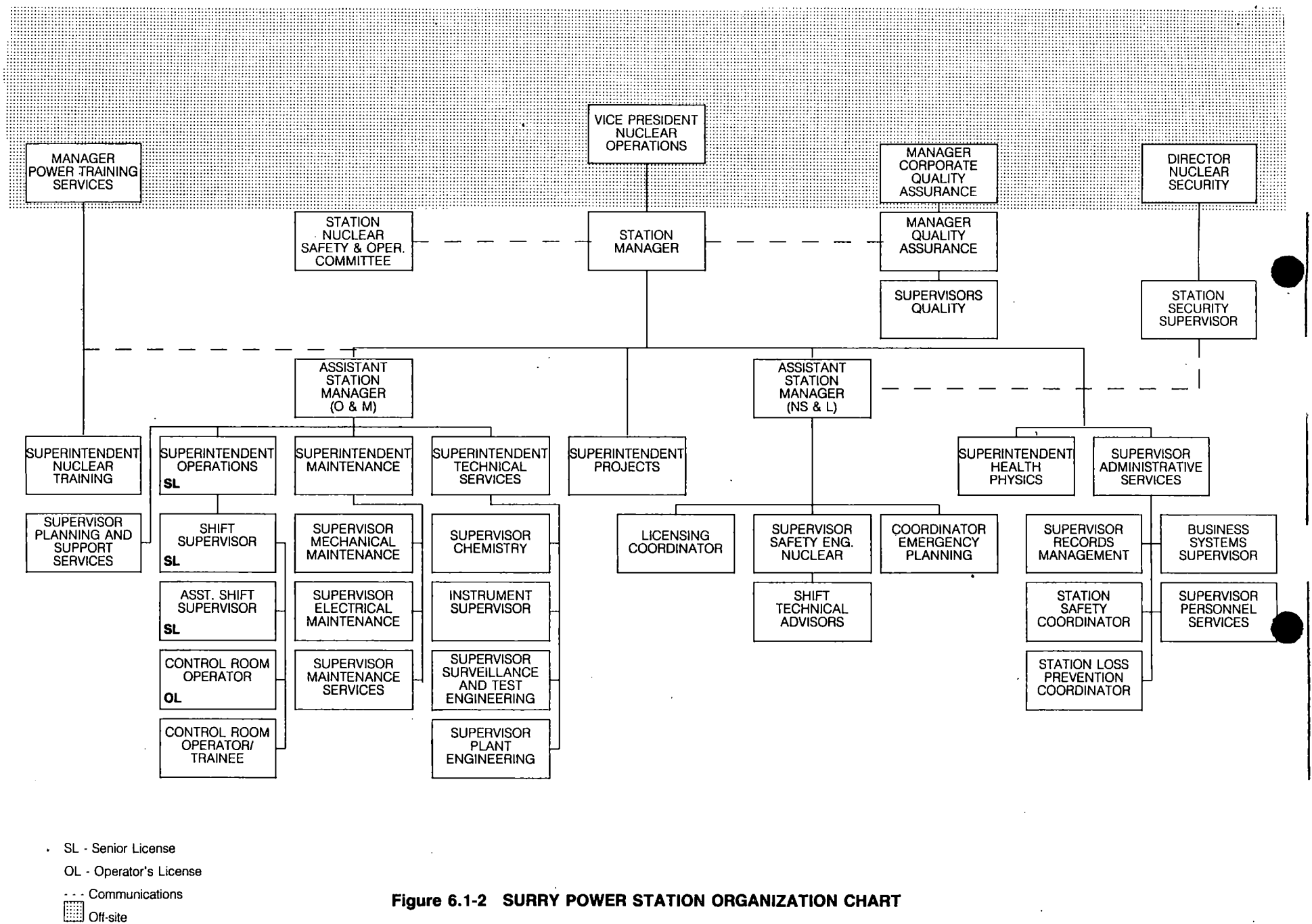


Figure 6.1-2 SURREY POWER STATION ORGANIZATION CHART

ATTACHMENT 2

SAFETY EVALUATION

**SURRY POWER STATION
UNIT NOS. 1 AND 2**

Description of Change

This proposed change will modify Section 6 of the Surry Technical Specifications to reflect a company reorganization in which the Quality Assurance organization will now report to the Senior Vice President - Engineering and Construction, rather than the Senior Vice President - Power Operations. It also changes the titles of several on-site and off-site managers and supervisors, and deletes the Director, Nuclear Training. As the major emphasis of the Company's nuclear program is on operations rather than construction, it is appropriate that the QA organization report to Engineering and Construction to enhance the independence of the QA organization.

Basis for No Significant Hazards Consideration

The proposed amendment does not involve a significant hazards consideration because operation of Surry Units 1 and 2 in accordance with the change would not:

- (1) involve a significant increase in the probability or consequences of an accident previously evaluated. This change merely revises where the QA organization reports to enhance independence and corrects titles in the on-site and off-site organization charts. Thus, this change does not change plant design or operation and does not increase the probability or consequences of an accident.
- (2) create the possibility of a new or different kind of accident from any previously analyzed. It has been determined that a new or different kind of accident will not be possible due to this change. Realigning the QA organization with Engineering and Construction, revising supervisor or manager titles or deleting the Director, Nuclear Training position do not create the possibility of a new or different kind of accident.
- (3) involve a significant reduction in a margin of safety. This change doesn't involve a change in the basis for any Technical Specification or the Updated Final Safety Analysis Report accident analysis. Therefore, the change doesn't involve a significant reduction in the margin of safety.

50.59 Safety Review

The proposed change does not involve a unreviewed safety question because operation of Surry Units 1 and 2 in accordance with this change would not:

- (1) increase the probability of occurrence or consequences of an accident or malfunction of equipment previously evaluated in the safety analysis report because these organizational changes and titles do not effect plant design or operations;
- (2) increase the possibility of an accident or malfunction of a different type than previously evaluated in the safety analysis report because the organization change does not effect plant design or operations.

- (3) reduce the margin of safety as defined in the basis for a Technical Specification because the change does not effect the basis of any Technical Specification or the Updated Final Safety Analysis Report safety analysis.

Pursuant to 10 CFR 50.59 and 50.92, it has been determined that this change does not involve a unreviewed safety question or a significant safety hazards consideration.