

VIRGINIA ELECTRIC AND POWER COMPANY  
RICHMOND, VIRGINIA 23261

W. L. STEWART  
VICE PRESIDENT  
NUCLEAR OPERATIONS

February 3, 1986

Mr. Harold R. Denton, Director  
Office of Nuclear Reactor Regulation  
Attn: Mr. Lester S. Rubenstein, Director  
PWR Project Directorate #2  
Division of PWR Licensing-A  
U.S. Nuclear Regulatory Commission  
Washington, D.C. 20555

Serial No. 86-040  
NO/dn  
Docket No. 50-280  
License No. DPR-32

Gentlemen:

VIRGINIA ELECTRIC AND POWER COMPANY  
SURRY POWER STATION UNIT NO. 1  
FIRST PERIOD, SECOND INTERVAL INSERVICE INSPECTION  
PROGRAM RELIEF REQUEST

Surry Power Station Unit No. 1 is scheduled to complete its first period, second interval inspection requirements for ASME Section XI, 1980 edition, Winter 1980 addenda, inspection program B, during the next refueling outage (May 1986). Pursuant to 10 CFR 50.55a(g)(5), we are requesting relief from certain code requirements defined in IWB 2412(b) of our code edition, for Surry Power Station Unit No. 1. The following basis is provided.

The code currently specifies in IEB 2412(b) that an inspection period may be decreased or extended by as much as a year to enable an inspection to coincide with a plant outage. Figure 1 provides an illustration of this code allowed tolerance band. Under this requirement Surry Unit 1 should complete all period 1, interval 2 inspection requirements by December 22, 1986, which allows the next refueling outage to be included into the first period inspection. Certain examinations, specifically involving the use of the reactor vessel examination device, are required to be completed within the first period of the second interval. These examinations are detailed on Attachment 1. Completion of these code required examinations places an impractical delay on the current (May 86) outage plan, affecting outage duration and power availability during "peak" season.

Alternatively, it is requested that these examinations (Attachment 1) be delayed to the following refueling outage. Additionally, second period, second interval examinations would be performed concurrent with the delayed examinations.

A similar request was submitted to extend the tolerance band for Surry Unit 2 (dated 6/14/83 from Mr. W. L. Stewart to Mr. H. R. Denton). This Relief Request was approved by letter (dated 5/7/84 from Mr. S. A. Varga to Mr. W. L. Stewart). The basis for the Surry Unit 2 exemption was the Steam Generator Replacement Program. A similar replacement program was

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PDR ADDOCK 05000288  
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completed for Surry Unit No. 1 between September 14, 1980 and July 6, 1981.

This relief request does not eliminate any examination. It only delays the requirements, extending the code tolerance band. As such, granting approval of this proposed relief would not endanger the public, nor affect the necessary inspection provided by the code.

In accordance with 10CFR170 an application fee of \$150 is enclosed.

If you have any questions, or need additional information to process this request please contact us.

Very truly yours,



W. L. Stewart

Attachment/Enclosure

cc: Dr. J. Nelson Grace  
Regional Administrator  
NRC Region II

Mr. Roger D. Walker, Director  
Division of Reactor Projects  
NRC Region II

Mr. Donald J. Burke  
NRC Resident Inspector  
Surry Power Station

Mr. Terence L. Chan  
NRC Surry Project Manager  
PWR Project Directorate #2  
Division of PWR Licensing-A

# ATTACHMENT 1

Category	Item	Description	Exam Method
B-A	B1.30	Reactor vessel shell to flange weld #1. Examine 50%.	UT
B-D	B3.90	Reactor vessel nozzle-to vessel welds #10, 12, 14. (outlet nozzles) Examine 100%.	UT
B-D	B3.100	Reactor Vessel nozzle inside radius sections. (outlet nozzles) Examine 100%.	UT
B-F	B5.10	Loop 1 outlet nozzle-to-safe-end weld #1DM. Examine 100%.	UT/PT
B-F	B5.10	Loop 3 outlet nozzle-to-safe-end weld #1DM. Examine 100%.	UT/PT
B-F	B5.10	Loop 2 outlet nozzle-to-safe-end weld #1DM. Examine 100%.	UT/PT

FIGURE 1

# ARTICLE IWB 2000 INSPECTION PROGRAM B

