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VICE PRESIDENT
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December 31, 1982

Mr. Harold R. Denton, Director
Office of Nuclear Reactor Regulation
Attn: Mr. Steven A. Varga, Chief
Operating Reactors Branch No. 1
Division of Licensing
U. S. Nuclear Regulatory Commission
Washington, D.C. 20555

Serial No. 728
PSE/KSB/mrh:0206C

Docket Nos. 50-280
50-281
License Nos. DPR-32
DPR-37

Gentlemen:

APPENDIX R TO 10CFR50, SECTION III.G.3
ALTERNATIVE SAFE SHUTDOWN
SURRY UNIT NOS. 1 AND 2

In the Surry Power Station Post Fire Safe Shutdown Review - Appendix "R" submitted by Vepco letter dated June 18, 1982, Serial No. 363 we identified several modifications required as a result of our review and indicated we would establish installation dates by January 1, 1983. This information was summarized in the "Commitments" section of the submittal.

This letter provides the required installation dates, which are discussed below:

1. Commitment 1 - The following additional instrumentation will be added to the remote monitoring panel. The circuits for this instrumentation will be routed through separate fire areas from the normal instrumentation.

   A. Steam Generator Pressure - Installation by 2nd refueling beginning after Jan. 1, 1982.

   B. Reactor Coolant Cold Leg Temperature - Installation by 2nd refueling beginning after Jan. 1, 1982.

   C. Source Range Neutron Flux Installation by the 2nd refueling beginning after Jan. 1, 1982.

   The above installation dates were previously provided in our Appendix "R" review.

2. Commitment 2 - Diesel generator control circuit to local panel. - Installation by 2nd refueling beginning after Juauary 1, 1982.

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3. Commitment 3 - Coordination of the 480V 225A frame size breakers - we are presently in process of reverification of the protective coordination data for the 480 load center and motor control center breakers. Upon completion of this activity, which includes receipt of vendor information, we will ensure that coordination does exist between the protective devices in question. We presently anticipate that we will receive all required information in sufficient time to enable required resetting to be completed by the 1st refuelings after January 1, 1983 on each unit, as previously stated in our Appendix "R" review. If equipment is required to replace existing equipment, it is anticipated that the lead time involved would require that installation occur in the same time frame as the other modifications.

4. Commitment 4 - Main breakers on Vital Bus Panels - replace with molded case switches by 2nd refuelings beginning after January 1, 1982.

5. Commitment 5 - Class 1E Motor Control Center 480/120V control transformers - replace with encapsulated transformers or fuse by 2nd refuelings beginning after January 1, 1982.

6. Commitment 6 - The additional Procedures required are:

   6a Complete lineup of the alternate method of charging - The line up of an alternate charging flow path is addressed in common Unit AP-46, Charging Pump Cross-Connect.

   6b Procedures to require breakers on MOVs 1700 and 1701 to be open when reactor coolant pressure requires the valves to be closed - OP-14, Residual Heat Removal System, Section 5.2, has been modified to include the opening of breakers for Unit 1 MOVs - 1700 and 1701, and Unit 2 MOV - 2700 and 2701 after valve closure.

   6c Emergency closure of the decay heat release valves - An open decay heat relief valve would, as with any other failed SG PORV or SV give steam break indications. Should such an event be sufficiently severe a Rx trip and EP-1 Reactor Trip are initiated. Further degredation could cause SI and EP-3, Loss of Secondary Coolant initiation. An event of this nature would be handled by existing steam break procedures.

6d Emergency closure of the pressurizer relief valves - An open PORV is detected by decreasing RCS pressure. This and other RCS pressure decreases are addressed in AP-42, Loss of Reactor Coolant Pressure. If pressure degrades sufficiently, a Rx trip occurs and EP-1, Reactor Trip is initiated. Further pressure reduction could cause SI and EP-2, Loss of Reactor Coolant initiation. An event of this nature would be handled by existing LOCA procedures.

6e Remote operation of Diesel Generators - Electrical adjustments loading of the EDGs can be performed only in the main CR. Local EDG engine operation is possible and is addressed by AP-17, Restoration and Loading of An Emergency Diesel Generator.

7. Commitment 7 - Charging Pump Service Water Pump Relocation - Installation by 1st refueling of Unit II after January 1, 1983, as previously committed to in our Appendix "R" review.

Therefore the installation for items 1, 2, 4, and 5 is scheduled during the refueling outages beginning on September 22, 1984 for Unit 1 and on October 19, 1984 for Unit 2. This installation schedule should be consistent with 10CFR Part 50.48(c)(4), which requires installation during the 1st refueling outage beginning at least 180 days after NRC approval (reference NRC letter Serial No. 667 dated November 18, 1982). The 180 day period ends May 17, 1983. Our upcoming refueling outages are scheduled to begin on February 1, 1983 for Unit 1 and on May 1, 1983 for Unit 2. Our engineering and procurement schedules are based on the 1984 refueling dates. We do not plan to adjust those schedules in the event that either of the upcoming refueling outages begin after May 17, 1983.

Should you need further information please feel free to contact us.

Very truly yours,

W. L. Stewart
Vice President
Nuclear Operations

cc: Mr. R. C. DeYoung, Director
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