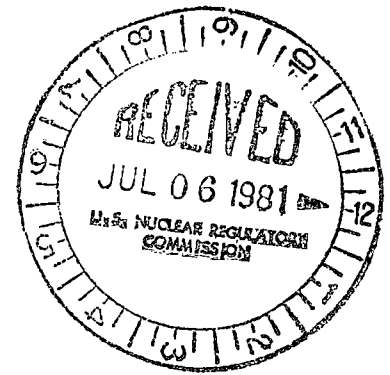


VIRGINIA ELECTRIC AND POWER COMPANY  
RICHMOND, VIRGINIA 23261

July 1, 1981

W. N. THOMAS  
VICE PRESIDENT  
FUEL RESOURCES



Mr. Harold R. Denton, Director  
Office of Nuclear Reactor Regulation  
Attn: Mr. Steven A. Varga, Chief  
Operating Reactors Branch No. 1  
Division of Licensing  
U.S. Nuclear Regulatory Commission  
Washington, D.C. 20555

Serial No.: 393  
FR/GLD: slm  
Docket No.: 50-280  
License No.: DPR-32

Dear Mr. Denton:

RELOAD INFORMATION FOR CYCLE 6  
SURRY NUCLEAR POWER STATION UNIT NO. 1

Surry Unit No. 1 completed its fifth cycle of operation on September 14, 1980 and went into an extended outage for steam generator replacement, refueling and other modifications. The purpose of this letter is to advise you of our plans for the Cycle 6 reload core.

The Cycle 6 reload core was analyzed in accordance with the methodology documented in Westinghouse Topical Report WCAP-9272 entitled "Westinghouse Reload Safety Evaluation Methodology." The analysis conducted was consistent with the plant modifications now being made on steam generators and safeguards systems (NPSH concerns). The results of this analysis indicated that no key analysis parameters would become more limiting during Cycle 6 operations than the values assumed in the currently applicable safety analysis. Further, the analysis demonstrated that the current Technical Specifications, as approved through Amendment No. 71, are appropriate and require no additional changes.

A detailed review of the Westinghouse methodology, analysis techniques and results has been conducted by our technical staff. In addition, a review has been performed by both the Station Nuclear Safety and Operating Committee and the Safety Evaluation and Control staff. It has been determined that no unreviewed safety questions as defined in 10CFR 50.59 will exist as a result of the Cycle 6 reload core.

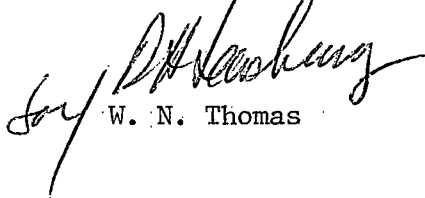
Verification of the reload core will be performed through a startup physics testing program. Unless otherwise indicated, this program will be consistent with documentation provided in our topical report VEP-FRD-36A, "Control Rod Reactivity Worth Determination by the Rod Swap Technique," transmitted by our letter to you dated January 16, 1981 (Serial No. 023). This report is a revision of VEP-FRD-36, an earlier topical report of the same title.

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*As of 5/10*

This letter is provided for your information. However, should you have questions, please contact us at your earliest convenience.

Very truly yours,

  
W. N. Thomas

cc: Mr. James P. O'Reilly, Director  
Office of Inspection and Enforcement  
Region II