

CONTROL BLOCK:

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 (1)

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

CON'T

0	1
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REPORT SOURCE

L	6	0	5	0	0	0	2	8	0	7	1	0	2	8	8	0	8	1	1	1	0	8	0	9
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60 61 DOCKET NUMBER 68 69 EVENT DATE 74 75 REPORT DATE 80

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0	8
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(17) LER/RO REPORT NUMBER EVENT YEAR
21 8 0 22

SEQUENTIAL REPORT NO. — 23

OCCURRENCE CODE 0 6 7 24 25 26

REPORT TYPE / 27

REVISION NO. 0 28 29 30 31 32

ACTION TAKEN		FUTURE ACTION		EFFECT ON PLANT		SHUTDOWN METHOD		HOURS				ATTACHMENT SUBMITTED		NPRD-4 FORM SUB.		PRIME COMP. SUPPLIER		COMPONENT MANUFACTURER			
D	18		19	Z	20	Z	21	0	0	0	0	Y	23	N	24	A	25	G	2	5	7
33	34	35	36	37	38	39	40	41	42	43	44	45	46	47	48	49	50	51	52	53	54

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1	3
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1 4 _____

FACILITY STATUS		% POWER			OTHER STATUS (30)		METHOD OF DISCOVERY		DISCOVERY DESCRIPTION (32)		
1	5	X	28	0	0	0	29	S/G Replacement	C	31	Special Examination.
7	8	9		10	11	12	13		45	46	80

ACTIVITY CONTENT
RELEASED OF RELEASE AMOUNT OF ACTIVITY (35)

1 6 Z (33) N/A 44

7 8 9 10 11

45 80

LOCATION OF RELEASE (36)

N/A

PERSONNEL EXPOSURES									
NUMBER			TYPE	DESCRIPTION					
1	7	0	0	0	(37)	Z	(38)	N/A	(39)

PERSONNEL INJURIES		NUMBER		DESCRIPTION	
1	3	0	0	0	N/A

1		2		3		4		5		6		7		8		9		10		11		12	
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[illegible]

NRC USE ONLY

ATTACHMENT 1
SURREY POWER STATION, UNIT 1
DOCKET NO: 50-280
REPORT NO: 80-067/01T-0
EVENT DATE: 10-28-80

MAIN STEAM PIPING WELD DISCREPANCIES

1. DESCRIPTION OF EVENT

With the Unit defueled, Radiography examination of the Main Steam elbow at the Steam Generator outlet nozzle revealed indications in the weld area. The indications were found on all three Steam Generators. This examination was being preformed as required by ETA 30006 in preparation for Steam Generator replacement. The indications are judged to be artifacts of the welding process and not service induced. This event is reportable per Technical Specification 6.6.2.a.(9).

2. PROBABLE CONSEQUENCES

The integrity of the Main Steam Piping may have been degraded. Since the Unit is in a defueled condition, the health and safety of the public was not affected.

3. CAUSE

The indications were the results of original construction of the main steam system.

4. IMMEDIATE CORRECTIVE ACTIONS

A review of the original construction records was initiated to determine the origin of these indications. The results of the review revealed that these indications did exist, and were originally judged to be acceptable.

5. SUBSEQUENT CORRECTIVE ACTIONS

The subject weld areas will be completely machined out and replaced during the Steam Generator Replacement.

6. ACTION TAKEN TO PREVENT RECURRENCE

None

7. GENERIC IMPLICATIONS

An evaluation of the original radiographs of the main steam piping inside the containment is in progress. Approximately 40% of the original radiographs have been evaluated and no rejectable indications have been identified. In addition, RT examination of some of the welds will be conducted to confirm the results of the above evaluation. The evaluation and RT examination will be completed prior to hydrostatic testing of the Steam Generators. A similar review of original construction radiographs for Unit No. 2 is in progress. It should be noted that the nozzle to elbow welds for unit No. 2 were replaced during the Steam Generator Outage.