

LICENSEE EVENT REPORT

EXHIBIT A

CONTROL BLOCK:

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9

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

7	0	1	8	9	V	A	S	P	S	1	14	2	0	0	-	0	0	0	0	0	-	0	0	25	3	4	1	1	1	1	4	57			58	5	
				LICENSEE CODE										LICENSE NUMBER										LICENSE TYPE													

CONT

REPORT SOURCE L 6 0 5 0 0 0 2 8 0 7 0 5 0 9 8 0 8 0 5 3 0 8 0 9

6C 61 DOCKET NUMBER 68 69 EVENT DATE 74 75 REPORT DATE 80

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0 2 During the performance of PT-16.4 (Containment Isolation Valve Leakage
0 3 with the unit at cold shutdown, the As Found leakage was determined to
0 4 exceed the allowable leakage rate specified in 10CFR50. This is contrary
0 5 to T.S.-4.4.A. Approximately 32% of the leakage from those valves of
0 6 concern are valves which open on a CLS actuation, therefore the leakage from
0 7 those valves would not have affected system integrity during an accident. The
0 8 remaining leakage would have been minimized so as not to affect the health
7 8 9 (cont'd on AS

(con'd on Att.1)

(cont'd on Att. 1)

0	9	SYSTEM CODE S	11	CAUSE CODE X	12	CAUSE SUBCODE Z	13	COMP. SUBCODE X	15	VALVE SUBCODE X	16
7	8	9	10	11	12	13	14	15	16	17	18
LER/RO REPORT NUMBER 17		EVENT YEAR 8 0		SEQUENTIAL REPORT NO. 0 2 5		OCCURRENCE CODE 0 3		REPORT TYPE L		REVISION NO. 0	
21	22	23	24	25	26	27	28	29	30	31	32
ACTION TAKEN B	FUTURE ACTION Z	EFFECT ON PLANT Z	SHUTDOWN METHOD Z		HOURS 0 0 0 0	ATTACHMENT SUBMITTED Y	NPRD-4 FORM SUB. N	PRIME COMP. SUPPLIER A	COMPONENT MANUFACTURER X 9 9 9		
33	34	35	36	37	38	39	40	41	42	43	44

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 0 Several valves of various types and sizes leaked excessively due to

1 1 normal wear from operational use. All valves that displayed excessive

1 2 leakage have been repaired or replaced, and retested.

1 3

1 4

7 8 9

1 5 G 28 0 0 0 29 NA 30 B 31 Normal testing 32

ACTIVITY		CONTENT		RELEASED		AMOUNT OF ACTIVITY		LOCATION OF RELEASE	
1	6	9	33	10	34	11	35	12	36
7	8	Z	Z	NA	NA				

PERSONNEL EXPOSURES									
NUMBER				TYPE	DESCRIPTION				
1	7	0	0	0	NA				

PERSONNEL INJURIES		NUMBER		DESCRIPTION	
1	2	0	0	0	NA

1		9		7		8		6		5		4		3		2		1	
LOSS OF OR DAMAGE TO FACILITY (43)																			
TYPE				DESCRIPTION															
Z (42)				NA															

7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60

PUBLICITY

ISSUED DESCRIPTION (45)

2 0 N NA

NRC USE ONLY

NAME OF PREPARER J. L. Wilson

PHONE: _____

8006030 348

ATTACHMENT 2
SURRY POWER STATION UNIT NO. 1
DOCKET NO: 50-280
REPORT NO: 80-025/03L-0
EVENT DATE: 05-09-80

TITLE OF REPORT: CONTAINMENT ISOLATION VALVES.

1. EVENT DESCRIPTION:

During the performance of PT-16.4 (Containment Isolation Valve Leakage) with the Unit at cold shutdown, it was determined that the As Found leakage rate was 1828.5 SCFH. This is greater than that specified in 10CFR50, and is contrary to T.S.-4.4.A.

2. PROBABLE CONSEQUENCES:

Approximately 32% of the leakage from those valves of concern are valves which open on CLS actuation, therefore the leakage from those valves would not have affected system integrity during an accident. The remaining leakage would have been minimized so as not to affect the health and safety of the general public with proper actuation of the Safety Injection and Emergency Core Cooling Systems.

3. CAUSE:

Due to the lengthy time between testing periods with additional down time for maintenance, several valves exhibited somewhat greater deterioration and wear than was expected from normal operational usage, thus causing more than allowable leakage.

4. IMMEDIATE CORRECTIVE ACTION:

As each valve, that demonstrated excess leakage, was found; a maintenance request (MR) was submitted to have the valve repaired.

5. SUBSEQUENT CORRECTIVE ACTION:

Following the submittals of the MR'S, the valves in question were repaired and retested. Subsequent testing verified all valves to be within specifications.

6. FUTURE CORRECTIVE ACTION:

Continue required testing per 10CFR50.

7. GENERIC IMPLICATIONS:

Because none of the valves displayed abnormal or unusual conditions considering the usage they received, there are no generic implications.

ATTACHMENT 1

SURRY POWER STATION, UNIT NO. 1

DOCKET NO: 50-280

REPORT NO: 80-025/03L-0

EVENT DATE: 05-09-80

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (CONTINUED)

and safety of the general public with proper actuation of the Safety Injection and Emergency Core Cooling Systems.