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50-281

VIRGINIA ELECTRIC AND POWER COMPANY  
RICHMOND, VIRGINIA 23261

December 17, 1979

Mr. James P. O'Reilly, Director  
Office of Inspection and Enforcement  
U. S. Nuclear Regulatory Commission  
Region II  
101 Marietta Street, Suite 3100  
Atlanta, Georgia 30303

Serial No. 865A/101779A.  
PO/FHT:scj  
Docket No. 50-281  
License No. DPR-37

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Dear Mr. O'Reilly:

Subject: IE Bulletin 79-13 Revision 2  
Surry Power Station Unit 2 Inspection Results

In response to the subject bulletin, "Cracking in Feedwater System Piping", Virginia Electric and Power Company herewith forwards the attached Report of Feedwater System Piping Inspections, Surry Power Station Unit 2.

If you have any questions or require additional information, please contact this office.

Very truly yours,

*W.C. Stallings for*  
C. M. Stallings

Vice President-Power Supply  
and Production Operations

cc: Director, Office of Inspection and Enforcement  
Division of Reactor Operations Inspection  
U. S. Nuclear Regulatory Commission  
Washington, D. C. 20555

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IE Bulletin 79-13, Revision 2REPORT OF FEEDWATER SYSTEM PIPING INSPECTIONSSURRY POWER STATION, UNIT NO. 2

In accordance with IE Bulletin 79-13, "Cracking in Feedwater System Piping", all feedwater nozzle-to-pipe areas and all feedwater pipe weld areas inside containment have been inspected on Surry Unit No. 2. Results of the inspection are as follows:

Steam Generator "A":

Circumferential crack indications were discovered in the base metal of the counterbore region on the reducer side of the reducer-to-nozzle weld. No crack indications were noted in the other sixteen welds inspected in the "A" Steam Generator Feedwater piping. One weld was rejected as follows:

W#7 - Small area of rejectable porosity.  
(Reported to NRC 8/23/79)

Steam Generator "B":

Circumferential crack indications were discovered in the base metal of the counterbore region of the reducer side of the reducer-to-nozzle weld. No rejectable indications of any type were discovered in the other fifteen welds inspected in the "B" Steam Generator Feedwater piping.

Steam Generator "C":

Incomplete fusion along with areas of porosity and slag were discovered in the reducer-to-nozzle weld. Weld was rejected. Linear indications were discovered in the root of the reducer-to-pipe weld. Weld was rejected. No rejectable indications of any type were discovered in the other fourteen welds inspected in the Steam Generator Feedwater piping.

It must be noted that several welds on the feedwater lines (three per line) were replaced as part of the Steam Generator Replacement Project. Since these welds were replaced prior to the IE Bulletin 79-13 implementation on Unit 2, NDE information on the original welds was unobtainable. The 3 new welds per feedwater line were radiographed to the SGRP construction code in force and not to the requirements of IE Bulletin 79-13.

Auxiliary Feedwater Connections:

The 3 in. auxiliary feedwater tie-in connections on all three main feedwater lines were magnetic particle inspected. The main feed lines were radiographed one pipe diameter immediately downstream of the auxiliary tie-ins. The results of these inspections were acceptable.

Visual inspection of all feedwater system piping supports and snubbers in containment has been completed to verify operability and conformance to design. No discrepancies were noted.

IE BULLETIN 79-13 RESPONSE: (CONT'D.)

Corrective actions for the items noted above included replacement of all three Steam Generator Feed reducers and weld repair to W#7, "A" SG. No further corrective actions are necessary.