NRC FORM 195 (2-76)

U.S. NUCLEAR REGULATORY COMMISSION

NRC DISTRIBUTION FOR PART 50 DOCKET MATERIAL

DOCKET NUMBER: 50-335

FILE NUMBER:

TO: Dennis L. Ziemann

FROM: FPL
Miami, FL. 33101
Robert E. Uhrig

DATE OF DOCUMENT: 03-21-77

DATE RECEIVED: 03-25-77

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DESCRIPTION

Ltr. Ref. our 01-17-77 ltr and their 10-17-77 ltr...Furnishing info regarding the ability of St. Lucie Unit #1 and others to accommodate a refueling accident within containment...

(2 pages) ACKNOWLEDGED

PLANT NAME: ST LUCIE UNIT # 1
jcm

DO NOT REMOVE

ENCLOSURE

FOR ACTION/INFORMATION

SAFETY		ENVIRO	
ASSIGNED AD:		ASSIGNED AD:	
✓ BRANCH CHIEF:	Ziemann (5)	BRANCH CHIEF:	
✓ PROJECT MANAGER:	Silver	PROJECT MANAGER:	
✓ LIC. ASST.:	Diggs	LIC. ASST.:	

INTERNAL DISTRIBUTION

✓	REG FILE		SYSTEMS SAFETY		PLANT SYSTEMS		SITE SAFETY &
✓	NRC PDR		HEINEMAN		TEDESCO		ENVIRO ANALYSIS
✓	I & E (2)		SCHROEDER		BENAROYA		DENTON & MULLER
✓	OELD				LAINAS		
✓	GOSSICK & STAFF		ENGINEERING		IPPOLITO		ENVIRO TECH.
	MIPC		MACARRY		KIRKWOOD		ERNST
	CASE		BOSNAK				BALLARD
	HANAUER		SIHWEIL		OPERATING REACTORS		YOUNGBLOOD
	HARLESS		PAWLICKI		STELLO		
							SITE TECH.
	PROJECT MANAGEMENT		REACTOR SAFETY		OPERATING TECH.		GAMMILL
	BOYD		ROSS	✓	EISENHUT		STEPP
	P. COLLINS		NOVAK	✓	SHAO		HULMAN
	HOUSTON		ROSZTOCZY	✓	BAER		
	PETERSON		CHECK	✓	BUTLER		SITE ANALYSIS
	MELTZ			✓	GRIMES		VOLLMER
	HELTEMES		AT & I				BUNCH
	SKOVHOLT		SALTZMAN			✓	J. COLLINS
			RUTBERG				KREGER

EXTERNAL DISTRIBUTION

✓	LPDR: Ft Pierce, Fla		NAT. LAB:		BROOKHAVEN NAT. LAB.
✓	TIC:		REG V.IE		ULRIKSON (ORNL)
✓	NSIC:		LA PDR		
	ASLB:		CONSULTANTS:		
✓	ACRS 16 CYS ~~HOLDING~~ SENT		AS CATD		

CONTROL NUMBER: 770870204 MAY

NRC FORM 195 (2-76)

FLORIDA POWER & LIGHT COMPANY

REGULATORY DOCKET FILE COPY

March 21, 1977
L-77-85

Office of Nuclear Reactor Regulation
Attn: Dennis L. Ziemann, Chief
Operating Reactors Branch #2
Division of Operating Reactors
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

RECEIVED
MAR 25 1977
U.S. NUCLEAR REGULATORY COMMISSION
Mail Section

Dear Mr. Ziemann,

Re: St. Lucie Unit 1
Docket No. 50-335
Postulated Refueling Accident

Your letter of 17 January 1977, relayed to us a concern expressed by Mr. Robert D. Pollard regarding the ability of St. Lucie Unit No. 1 and others to accommodate a refueling accident within containment. This issue was previously reviewed by the Staff during the development of the St Lucie Unit No. 1 Technical Specifications. (See Florida Power & Light Company letter L-75-506 dated 17 October 1975, at item 42 of the attachment.) The upper limit dose was evaluated and provided at that time. The maximum thyroid dose is 40 rem and the corresponding whole body dose is 0.17 rem, which are small fractions of the 10 CFR 100 guideline limits of 300 rem thyroid and 25 rem whole body. The principal assumptions used in this evaluation were in accordance with Regulatory Guide 1.25 and are as follows:

i) A rupture of an entire fuel assembly and release of the gap inventory in accordance with RG 1.25 was assumed.

ii) The effective DF of shielding water was 100 for iodine and 1 for Kr-85.

iii) No credit for dilution or mixing within the containment or reactor shield building annulus was assumed.

iv) No credit for charcoal filters was assumed.

v) Containment isolation was not assumed.

vi) The 2-hour accident meterological dilution factor was assumed.

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Since the bounding evaluation assumed that radionuclides emerging from the pool are transported immediately to the receptor under adverse meterological conditions with the resultant doses being small fractions of 10 CFR 100 guideline values, and a more realistic evaluation would yield significantly smaller doses, Mr. Pollard's concern is clearly not relevant to the St. Lucie facility. Accordingly, we believe further evaluation of this issue is unwarranted. If you require any additional clarifying data, we would be pleased to furnish it.

Yours very truly,

Robert E. Uhrig

Robert E. Uhrig
Vice President

REU/LLL/bb