

C 02/07/78

REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)
DISTRIBUTION FOR INCOMING MATERIAL

50-335

REC: OREILLY J P
NRC

ORG: SCHMIDT A D
FL PWR & LIGHT

DOCDATE: 06/09/78
DATE RCVD: 07/05/78

DOCTYPE: LETTER NOTARIZED: NO

COPIES RECEIVED

SUBJECT:

LTR 1 ENCL 1

LICENSEE EVENT REPT #78-016 ON 5/26/78 CONCERNING CONTAINMENT PRESSURE-HIGH
TRIP SETPOINT (TABLE 2.2-1) APPEARED TO BE IN ERROR DURING THE REFUELING
OUTAGE.

PLANT NAME: ST LUCIE #1

REVIEWER INITIAL: XRS
DISTRIBUTOR INITIAL: *ne*

***** DISTRIBUTION OF THIS MATERIAL IS AS FOLLOWS *****

INCIDENT REPORTS
(DISTRIBUTION CODE A002)

FOR ACTION: BR CHIEF ORB#4 BC**W/4 ENCL

INTERNAL:

REG FILE**W/ENCL
~~I & E**W/2 ENCL~~
I & C SYSTEMS BR**W/ENCL
NOVAK/CHECK**W/ENCL
AD FOR ENG**W/ENCL
HANAUER**W/ENCL
AD FOR SYS & PROJ**W/ENCL
ENGINEERING BR**W/ENCL
KREGER/J, COLLINS**W/ENCL
K SEYFRIT/IE**W/ENCL

NRC PDR**W/ENCL
MIPC**W/3 ENCL
EMERGENCY PLAN BR**W/ENCL
EEB**W/ENCL
PLANT SYSTEMS BR**W/ENCL
AD FOR PLANT SYSTEMS**W/ENCL
REACTOR SAFETY BR**W/ENCL
VOLLMER/BUNCH**W/ENCL
POWER SYS BR**W/ENCL

EXTERNAL:

LPDR'S
FT PIERCE, FL**W/ENCL
TIC**W/ENCL
NSIC**W/ENCL
ACRS CAT B**W/16 ENCL

A/04

DISTRIBUTION: LTR 45 ENCL 45
SIZE: 1P+1P+1P

CONTROL NBR: 781870322

***** THE END *****

GP

*** MASTER ROSTER ***
*** PUBLICATIONS ***

VISION	I-----	DISTRIBUTION	CODES	1 - 17	-----I
FILE	I-----	DISTRIBUTION	QTY	1 - 17	-----I
TY	I-----	DISTRIBUTION	CODES	18 - 34	-----I
ST..ZIP	I-----	DISTRIBUTION	QTY	18 - 34	-----I
P. TYPE	I-----	DISTRIBUTION	CODES	35 - 50	-----I
	I-----	DISTRIBUTION	QTY	35 - 50	-----I

	SN	SR	SF	SE	SM	SP	ST	SO	SA	SG
KLAND	1	1	1	1	1	1	1	1	1	1
CA	94621									

	CW
KLAND	1
CA	94621

V RAYMOND INTERNATIONAL	SN	SR	SF	SE	SM	ST	SG
KLAND	1	1	1	1	1	1	1
CA	94623						

VANCED TECH DIV-NUCLEAR	SN	SR	SF	SE	SM	ST	SO	SG
JECT MANAGER	1	1	1	1	1	1	1	1
KLAND	1	1	1	1	1	1	1	1
CA	94623							

WER INFO CENTER KB 16	SN	SR	SF	SE	SM	SP	ST	SO	SA	SG	NB
KLAND	1	1	1	1	1	1	1	1	1	1	1
CA	94623										

WER DIVISION	SN	SR	SF	SE	SM	SP	ST	SO	SA	SG	R1	R2	R3	R4	R5
KLAND	1	1	1	1	1	1	1	1	1	1	1	1	1	1	1
CA	94623														

ISER CENTER	SN	SE	SM	SG
KLAND	1	1	1	1
CA	94666			

OWER DIVISION	SN	SF	SE	SM	SO	N4
KLAND	1	1	1	1	1	1
CA	94666					

ANAGER LICENSING SERVICES	N4
KLAND	1
CA	94666

ECTOR MARKETING DEPARTMENT	CW
LEY	1
CA	94703

	SN	SF	SE	SM	SP	ST	SO	SG
KELEY	1	1	1	1	1	1	1	1
CA	94704							

REGULATORY DOCKET FILE COPY



June 9, 1978

PRN-LI-78-153

Mr. James P. O'Reilly, Director, Region II
Office of Inspection and Enforcement
U. S. Nuclear Regulatory Commission
230 Peachtree Street, N. W., Suite 1217
Atlanta, Georgia 30303

Dear Mr. O'Reilly:

REPORTABLE OCCURRENCE 335-78-16
ST. LUCIE UNIT 1
DATE OF OCCURRENCE: MAY 26, 1978

TECHNICAL SPECIFICATION 2.2.1
CONTAINMENT PRESSURE TRIP SETPOINT

The attached Licensee Event Report is being submitted in accordance with Technical Specification 6.9 to provide prompt notification of the subject occurrence.

Very truly yours,

A. D. Schmidt
Vice President
Power Resources

MAS/cpc

Attachment

cc: Robert Lowenstein, Esquire
Director, Office of Inspection and Enforcement (40)
Director, Office of Management Information and
Program Control (3)

781870322

CONTROL BLOCK: 1 1 1 1 1 1 1 (1) (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

F I L E S I T		(2) 0101-101010101010		(3) 41111111		(4) 1		(5)	
L I C E N S E E C O D E		L I C E N S E N U M B E R		L I C E N S E T Y P E		C A T			
CON'T		REPORT SOURCE		DOCKET NUMBER		EVENT DATE		REPORT DATE	
(01)		X(6) 015101010131315		(7) 01512161718		(8) 01610191718		(9)	
E V E N T D E S C R I P T I O N A N D P R O B A B L E C O N S E Q U E N C E S (10)									
(02) Shortly after Cycle 1 shutdown, the NSSS vendor noted that the "contain-									
(03) ment pressure-high" trip setpoint (T.S. Table 2.2-1) appeared to be in error.									
(04) During the refueling outage, the Specification was confirmed to be non-									
(05) conservative. The correct Specification has been determined and has been in									
(06) use since the beginning of Cycle 2.									
(07)									
(08)									
SYSTEM CODE CAUSE CODE CAUSE SUBCODE COMPONENT CODE COMP. SUBCODE VALVE SUBCODE									
(09) I A (11)		A (12)		X (13)		Z Z Z Z Z Z Z (14)		Z (15)	
LEI/RO REPORT NUMBER		EVENT YEAR		SEQUENTIAL REPORT NO.		OCCURRENCE CODE		REPORT TYPE	
(17) 78		[]		0116		011		T	
ACTION TAKEN		FUTURE ACTION		EFFECT ON PLANT		SHUTDOWN METHOD		HOURS	
(18) E		(19) Z		(20) Z		(21) Z		(22) 0101010	
ATTACHMENT SUBMITTED		NPRD-1 FORM SUB.		PRIME COMP. SUPPLIER		COMPONENT MANUFACTURER		REVISION NO.	
(23) Y		(24) N		(25) Z		(26) Z 9 9 9		(27) 0	
CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)									
(10) A vendor recommended setpoint change was incorporated into the Final Safety									
(11) Analysis Report, but for unknown reasons (presumably personnel error) it									
(12) was not incorporated in the Technical Specifications. A Technical Specifica-									
(13) tion change request is being prepared to correct Table 2.2-1.									
(14)									
FACILITY STATUS % POWER OTHER STATUS METHOD OF DISCOVERY DISCOVERY DESCRIPTION									
(15) H (28)		01010 (29)		NA (30)		ID (31)		Notification from NSSS (32)	
ACTIVITY TAKEN		CONTENT RELEASED OR RELEASE		AMOUNT OF ACTIVITY		LOCATION OF RELEASE			
(16) Z (33)		Z (33)		NA (35)		NA (36)			
PERSONNEL EXPOSURES NUMBER		TYPE		DESCRIPTION					
(17) 01010 (37)		Z (38)		NA (39)					
PERSONNEL INJURIES NUMBER		DESCRIPTION							
(18) 01010 (40)				NA (41)					
LOSS OF OR DAMAGE TO FACILITY TYPE		DESCRIPTION							
(19) Z (42)				NA (43)					
PUBLICITY ISSUED DESCRIPTION									
(20) N (44)				NA (45)					
NAME OF PREPARER M. A. Schoppman PHONE 305/552-3802									

Additional Event Description

Shortly after Cycle 1 shutdown, the NSSS vendor noted (in a letter regarding core Cycle 2 Specifications) that the reactor protective instrumentation trip setpoint for "containment pressure - high" (T. S. Table 2.2-1) appeared to be in error. During the refueling outage, records were reviewed and it was found that the vendor had recommended a setpoint change shortly before St. Lucie Unit 1 was licensed, but the change had not been incorporated into the Technical Specifications. The setpoint in Table 2.2-1 is ≤ 3.9 psig, and the actual Cycle 1 operating setpoint was "conservatively" set at ≤ 3.5 psig. The vendor was requested to provide further information, and, as a precaution before Cycle 2 reactor startup after refueling, the actual operating setpoint was reduced to ≤ 3.3 psig (as recommended by the vendor). On May 26, 1978, the vendor confirmed that the required setpoint is ≤ 3.3 psig.