D 08/02/78

REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)
DISTRIBUTION FOR INCOMING MATERIAL 50-335

REC: OREILLY J P
NRC

ORG: SCHMIDT A D
FL PWR & LIGHT

DOCDATE: 07/07/78
DATE RCVD: 07/31/78

DOCTYPE: LETTER NOTARIZED: NO
SUBJECT:
FORWARDING LICENSEE EVENT REPT (RO 50-335/78-020) ON 06/07/78 CONCERNING DURING PWR ASCENSION PHYSICS TESTING FOLLOWING A REFUELING OUTAGE, CEA #65 DROPPED... W/ATT.

COPIES RECEIVED
LTR 1 ENCL 1

PLANT NAME: ST LUCIE #1

REVIEWER INITIAL: XJM
DISTRIBUTOR INITIAL: DL

********** DISTRIBUTION OF THIS MATERIAL IS AS FOLLOWS **********

INCIDENT REPORTS
(DISTRIBUTION CODE A002)

FOR ACTION: BR CHIEF ORB#4 BC**W/4 ENCL

INTERNAL:
REG FILE**W/ENCL
I & E**W/2 ENCL
I & C SYSTEMS BR**W/ENCL
NOVAK/CHECK**W/ENCL
AD FOR ENG**W/ENCL
HANAUER**W/ENCL
AD FOR SYS & PROJ**W/ENCL
ENGINEERING BR**W/ENCL
KREGER/J. COLLINS**W/ENCL
K SEYFRIT/IE**W/ENCL
NRC PDR**W/ENCL
MIPC**W/3 ENCL
EMERGENCY PLAN BR**W/ENCL
EEB**W/ENCL
PLANT SYSTEMS BR**W/ENCL
AD FOR PLANT SYSTEMS**W/ENCL
REACTOR SAFETY BR**W/ENCL
VOLLMER/BUNCH**W/ENCL
POWER SYS BR**W/ENCL

EXTERNAL:
LPDR'S
FT PIERCE, FL**W/ENCL
TIC, LIZ CARTER**W/ENCL
NSIC**W/ENCL
ACRS CAT B**W/16 ENCL

DISTRIBUTION: LTR 45 ENCL 45
SIZE: 1P+1P+1P

CONTROL NBR: 782140109

AD GD 4

********** THE END **********

FPL
FLORIDA POWER & LIGHT COMPANY

July 7, 1978

PRN-LI-78-180

Mr. James P. O'Reilly, Director, Region II
Office of Inspection and Enforcement
U. S. Nuclear Regulatory Commission
230 Peachtree Street, N. W., Suite 1217
Atlanta, Georgia 30303

Dear Mr. O'Reilly:

REPORTABLE OCCURRENCE 335-78-20
ST. LUCIE UNIT 1
DATE OF OCCURRENCE: JUNE 7, 1978

TECHNICAL SPECIFICATION 3.1.3.1.e
CEA 65

The attached Licensee Event Report is being submitted in accordance with Technical Specification 6.9 to provide 30-day notification of the subject occurrence.

Very truly yours,

A. D. Schmidt
Vice President
Power Resources

MAS/cpc

Attachment

cc: Harold F. Reis, Esquire
Director, Office of Inspection and Enforcement (30)
Director, Office of Management Information and
Program Control (3)

A002
S/1

NRC FORM 366
(7-77)

U. S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT

CONTROL BLOCK: | | | | | | | (1) (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

01 | F L S L S 1 (2) | 0 0 - 0 0 0 0 0 - 0 0 (3) | 4 1 1 1 1 (4) | (5)
LICENSEE CODE — LICENSE NUMBER — LICENSE TYPE — CAT

CON'T 01 | REPORT SOURCE L (6) | 0 5 0 0 0 3 3 5 (7) | 0 6 0 7 7 8 (8) | 0 7 0 7 7 8 (9)
DOCKET NUMBER — EVENT DATE — REPORT DATE

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

02 During power ascension physics testing following a refueling outage, CEA
03 #65 dropped. Reactor power was about 80 percent and the CEA group, of
04 which CEA #65 was a part, was in the process of being inserted manually
05 for testing. Required action in accordance with Technical Specification
06 3.1.3.1.e was taken and CEA #65 was restored to its normal position within
07 the applicable time limit.
08

	SYSTEM CODE	CAUSE CODE	CAUSE SUBCODE	COMPONENT CODE	COMP. SUBCODE	VALVE SUBCODE
09	R B (11)	X (12)	X (13)	Z Z Z Z Z Z (14)	X (15)	Z (16)

(17) LER/RO REPORT NUMBER

EVENT YEAR		SEQUENTIAL REPORT NO.		OCCURRENCE CODE	REPORT TYPE		REVISION NO.
7 8	—	0 2 0	/	0 3	L	—	0

ACTION TAKEN	FUTURE ACTION	EFFECT ON PLANT	SHUTDOWN METHOD	HOURS (22)	ATTACHMENT SUBMITTED	NPRD-4 FORM SUB.	PRIME COMP. SUPPLIER	COMPONENT MANUFACTURER
Z (18)	Z (19)	B (20)	Z (21)	0 0 0 1	Y (23)	N (24)	Z (25)	Z 9 9 9 (26)

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

10 The cause of the dropped CEA is not known. No cause could be determined
11 for tripping and no other abnormal conditions were found. CEA #65
12 operated properly when restored to its normal position and has performed
13 normally for four weeks.
14

	FACILITY STATUS	% POWER	OTHER STATUS (30)	METHOD OF DISCOVERY	DISCOVERY DESCRIPTION (32)
15	B (28)	0 8 0 (29)	NA	A (31)	Operator Observation

	ACTIVITY RELEASED	CONTENT OF RELEASE	AMOUNT OF ACTIVITY (35)	LOCATION OF RELEASE (36)
16	Z (33)	Z (34)	NA	NA

PERSONNEL EXPOSURES

	NUMBER	TYPE	DESCRIPTION (39)
17	0 0 0 (37)	Z (38)	NA

PERSONNEL INJURIES

	NUMBER	DESCRIPTION (41)
18	0 0 0 (40)	NA

LOSS OF OR DAMAGE TO FACILITY (43)

	TYPE	DESCRIPTION
19	Z (42)	NA

PUBLICITY

	ISSUED	DESCRIPTION (45)	NRC USE ONLY
20	N (44)	NA	

NAME OF PREPARER M. A. Schoppman

PHONE: (305) 552-3802

Additional Event Description

As a result of the reactivity insertion, azimuthal power tilt (T_q) increased to 0.076 and total integrated radial peaking factor (F_r^t) increased to 1.576, which required action in accordance with Technical Specifications 3.2.4 and 3.2.3, respectively. Reactor power was reduced, and T_q and F_r^t were returned to normal within the applicable time limits. This was the third occurrence of a dropped CEA for which the cause is not known (refer to LER 335-77-6 and LER 335-78-18).