

## LICENSEE EVENT REPORT

CONTROL BLOCK:

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

01 V A S P S 1 2 0 0 - 0 0 0 0 0 - 0 0 3 4 1 1 1 1 4 5  
7 8 9 14 15 25 26 30 57 CAT 58

CON'T

01 REPORT SOURCE L 6 0 5 0 0 0 2 8 0 7 1 2 2 0 7 8 8 0 1 0 2 7 9 9  
7 8 60 61 DOCKET NUMBER 68 69 EVENT DATE 74 75 REPORT DATE 80

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

02 On 12-20-78 Westinghouse notified Vepco of an input error in the currently applicable  
03 LOCA-ECCS analysis. The error was estimated to result in an increase in peak clad  
04 temperature of approximately 20°F. Further investigation revealed that the analysis  
05 input was correct but a modeling methodology which was slightly different from the  
06 methodology applied in the NRC approved Westinghouse ECCS Evaluation Model (Feb. 1978  
07 version) was being used. This is reportable per Tech. Spec. 6.6.2.a.(8). The  
08 health and safety of the public were not affected.

09 SYSTEM CODE CAUSE CODE CAUSE SUBCODE COMPONENT CODE COMP. SUBCODE VALVE SUBCODE  
X X 11 X 12 X 13 Z Z Z Z Z Z 14 Z 15 Z 16  
7 8 9 10 11 12 13 18 19 20

17 LER/RO REPORT NUMBER 7 8 21 22  
18 ACTION TAKEN X 19 FUTURE ACTION  
20 EFFECT ON PLANT Z 21 SHUTDOWN METHOD Z 22 HOURS 0 0 0 0  
23 SEQUENTIAL REPORT NO. 0 5 0 24 26  
25 OCCURRENCE CODE 0 1 27 28  
29 REPORT TYPE T 30  
31 REVISION NO. 0 32

33 X 34 X 35 Z 36 Z 37 0 0 0 0 40  
41 Y 42 N 43 Z 44 Z 9 9 9 26 47

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

10 The margin to the Technical Specification limit on  $F_Q$  and to the 10 CFR 50.46 limits  
11 was assessed and found to be sufficiently conservative to accommodate the above non-  
12 conservatism. A new analysis with the February 1978 Westinghouse LOCA-ECCS evaluation  
13 model, which includes more conservative modeling methodology was performed and has  
14 been submitted to the Nuclear Regulatory Commission for approval.

15 FACILITY STATUS G 28 % POWER 0 0 0 29 OTHER STATUS NA 30 METHOD OF DISCOVERY D 31 DISCOVERY DESCRIPTION NSS Vendor Notification 32  
7 8 9 10 12 13 44 45 46 80

16 ACTIVITY RELEASED Z 33 CONTENT OF RELEASE Z 34 AMOUNT OF ACTIVITY NA 35 LOCATION OF RELEASE NA 36  
7 8 9 10 11 44 45 80

17 PERSONNEL EXPOSURES NUMBER 0 0 0 37 TYPE Z 38 DESCRIPTION NA 39  
7 8 9 11 12 13 80

18 PERSONNEL INJURIES NUMBER 0 0 0 40 DESCRIPTION NA 41  
7 8 9 11 12 80

19 LOSS OF OR DAMAGE TO FACILITY TYPE Z 42 DESCRIPTION NA 43  
7 8 9 10 80

20 PUBLICITY ISSUED N 44 DESCRIPTION NA 45  
7 8 9 10 80

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Surry Power Station, Unit 1  
Docket No: 50-280  
Report No: 78-050/01T-0  
Title of Report: LOCA-ECCS Analysis

1. Description of Event:

On 12-20-78 Westinghouse notified Vepco of an input error in the currently applicable LOCA-ECCS analysis. The error was estimated to result in an increase in peak clad temperature of approximately 20°F. Further investigation revealed that the analysis input was correct but a modeling methodology which was slightly different from the methodology applied in the NRC approved Westinghouse ECCS Evaluation Model (Feb. 1978 version) was being used.

2. Probable Consequences:

Because there is sufficient margin in the currently applicable LOCA-ECCS to accomodate this non-conservatism, there are no probable consequences of this event.

3. Cause of Event:

The cause of this event is an alternate modeling methodology used in analyzing containment pressure response effects resulting from the broken loop accumulator flow.

4. Immediate Corrective Actions:

The margin to the Technical Specification Limit on  $F_Q$  and the 10 CFR 50.46 limits was assessed and found to be sufficiently conservative to accomodate the above non-conservatism.

5. Subsequent Corrective Actions:

A new analysis performed with the February 1978 Westinghouse LOCA-ECCS evaluation model, which includes more conservative modeling methodology, has been submitted to the Nuclear Regulatory Commission for approval.

6. Actions Taken to Prevent Recurrence:

None necessary.

7. Generic Implications:

None.