

50-335

NRC DISTRIBUTION FOR PART 50 DOCKET MATERIAL

FILE NUMBER

INCIDENT REPORT

TO: Mr Moseley

FROM: Florida Pwr & Light Co
Miami, Fla
R E Uhrig

DATE OF DOCUMENT

4-23-76

DATE RECEIVED 5-5-76

☒ LETTER☐ NOTORIZED

PROP

INPUT FORM

NUMBER OF COPIES RECEIVED

☐ ORIGINAL☒ UNCLASSIFIED☒ COPY

none signed

DESCRIPTION

Ltr trans the following license event report:

ENCLOSURE

RO# 76-7 on 3-25-76 concerning erratic reactor
coolant loop Th temperature indication.....

DO NOT REMOVE

AS KNOWLEDGED

PLANT NAME:

St Lucie #1

NOTE: IF PERSONNEL EXPOSURE IS INVOLVED
SEND DIRECTLY TO KREGER/J. COLLINS

SAFETY

FOR ACTION/INFORMATION

ENVIRO 5-10-76

ehf

BRANCH CHIEF:

Ziemann

W/3 CYS FOR ACTION

LIC. ASST:

Diggs

W/1 CYS

ACRS 16 CYS HOLDING/SENT TO LA

INTERNAL DISTRIBUTION

REG FILE

NRC PDR

I & E (2)

MIPC (3)

SCHROEDER/IPPOLITO

HOUSTON

NOVAK/CHECK

GRIMES/SCHWENCER

CASE

F. WILLIAMS

HANAUER

TEDESCO/MACCARY

EISENHUT

BAER

SHAO

VOILLMER/BUNCH

KREGER/J. COLLINS

EXTERNAL DISTRIBUTION

LPDR: Ft Pierce, Fla

TIC

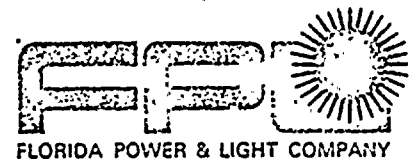
NSIC

CONTROL NUMBER

4469

12

27



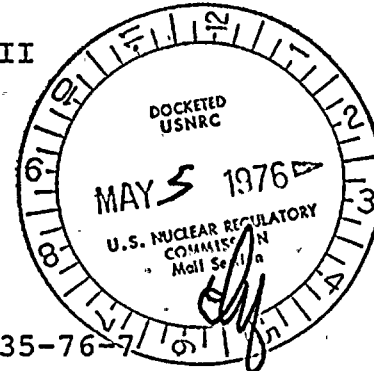
April 23, 1976

PRN-LI-76-97

50-335

Mr. Norman C. Moseley, Director, Region II
Office of Inspection and Enforcement
U. S. Nuclear Regulatory Commission
230 Peachtree Street, N. W., Suite 818
Atlanta, Georgia 30303

Dear Mr. Moseley:



REPORTABLE OCCURRENCE 335-76-7
ST. LUCIE UNIT 1
DATE OF OCCURRENCE: MARCH 25, 1976

WIRING DIAGRAM ERROR
(Th INSTRUMENT)

The attached Licensee Event Report is being submitted in accordance with Technical Specification 6.9 to provide 30-day notification of the subject occurrence.

Very truly yours,

A. D. Schmidt

A. D. Schmidt
Vice President
Power Resources

REGULATORY DOCKET FILE COPY

MAS/cpc

Attachment

cc: Jack R. Newman, Esquire
Director, Office of Inspection and Enforcement (30)
Director, Office of Management Information and
Program Control (3)

4469

LICENSEE EVENT REPORT

CONTROL BLOCK: 1 6

[PLEASE PRINT ALL REQUIRED INFORMATION]

LICENSEE NAME														LICENSE NUMBER														LICENSE TYPE										EVENT TYPE			
01	F	L	S	L	S	1									0	0	-	0	0	0	0	-	0	0					4	1	1	1	1					0	3		
7	8	9				14									15									25	26					30					31	32					

CATEGORY				REPORT TYPE		REPORT SOURCE		DOCKET NUMBER										EVENT DATE										REPORT DATE									
01	CONT					L	L	0	5	0	-	0	3	3	5					0	3	2	5	7	6					0	4	2	3	7	6		
7	8				57	58	59	60	61									68	69					74	75					80							

EVENT DESCRIPTION

02	While performing preoperational testing of the Reactor Protection System																																																																																80
03	(RPS), erratic Reactor Coolant Loop Th temperature indication was																																																																																80
04	observed. Subsequent review of Reactor Coolant System (RCS) loop																																																																																80
05	temperature wiring diagrams revealed an error. The polarity of the																																																																																80
06	average Th signal from the loops to the Core Protection Calculator for																																																																																80

SYSTEM CODE				CAUSE CODE		COMPONENT CODE										PRIME COMPONENT SUPPLIER				COMPONENT MANUFACTURER										VIOLATION									
07	I	A			B									I	N	S	T	R	U					N					G	0	8	0					N		
7	8	9	10			11									17					43					44					47					48				

CAUSE DESCRIPTION

08	This occurrence was caused by an error in the RCS loop temperature																																																																																80
09	wiring diagrams. Corrective action has been as described in the Event																																																																																80
10	Description.																																																																																80

FACILITY STATUS				% POWER				OTHER STATUS										METHOD OF DISCOVERY				DISCOVERY DESCRIPTION																			
11	B			0	0	0					NA					C				Preoperational Test																					
7	8	9			10	11	12	13					44	45				46																							

FORM OF ACTIVITY RELEASED				CONTENT OF RELEASE				AMOUNT OF ACTIVITY										LOCATION OF RELEASE																			
12	Z			Z				NA										NA																			
7	8	9			10	11					44	45																									

PERSONNEL EXPOSURES

NUMBER				TYPE		DESCRIPTION										
13	0	0	0	Z	NA											
7	8	9			11	12										

PERSONNEL INJURIES

NUMBER				DESCRIPTION												
14	0	0	0	NA												
7	8	9			11	12										

PROBABLE CONSEQUENCES

15	NA																																																																																80
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LOSS OR DAMAGE TO FACILITY

TYPE				DESCRIPTION											
16	Z	NA													
7	8	9			10										

PUBLICITY

17	NA																																																																																80
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ADDITIONAL FACTORS

18	See Page Two for continuation of Event Description.																																																																																80
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19																																																																																	80
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NAME: M. A. Schoppman

PHONE: (305) 552-3779

REPORTABLE OCCURRENCE 335-76-7
LICENSEE EVENT REPORT
PAGE TWO

Event Description (Continued)

the Thermal Margin/Low Pressure reactor trip was found reversed on the diagrams. Component modifications were completed prior to continuing heatup to within the T_h instrument range. The modifications were made to comply with the original design intent, and no functions or equipment were added or deleted. Further preoperational testing and reviews of RPS wiring have indicated that this is not a repetitive problem. This was the first occurrence of this type. (335-76-7).