

50-335

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FILE NUMBER

TO: Mr Stello

FROM: Florida Pwr & Light Co
Miami, Fla
R E UhrigDATE OF DOCUMENT
4-16-76DATE RECEIVED
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☐ COPY☒ NOTORIZED
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PROP

INPUT FORM

NUMBER OF COPIES RECEIVED

3 signed

DESCRIPTION

Ltr notarized 3-16-76....trans the following:

ENCLOSURE

Amdt to OL/Change to tech specs with regard
to deletion of special testing exception
pertaining to control rod withdrawal.....

(43 cys encl rec'd)

ACKNOWLEDGED

DO NOT REMOVE

PLANT NAME: St Lucie #1

SAFETY

FOR ACTION/INFORMATION

ENVIRO

4-20-76

ehf

ASSIGNED AD :

BRANCH CHIEF :

PROJECT MANAGER:

LIC. ASST. :

Ziemann (5)

Sylvester

Diss

ASSIGNED AD :

BRANCH CHIEF :

PROJECT MANAGER :

LIC. ASST. :

INTERNAL DISTRIBUTION

REG FILE

NRC PDR

I & E (2)

OELD

GOSSICK & STAFF

MIPC

CASE

HANAUER

HARLESS

SYSTEMS SAFETY

HEINEMAN

SCHROEDER

ENGINEERING

MACCARY

KNIGHT

SIHWEIL

PAWLICKI

PLANT SYSTEMS

TEDESCO

BENAROYA

LAINAS

IPPOLITO

OPERATING REACTORS

STELLO

OPERATING TECH

EISENHUT

SHAO

BAER

SCHWENCER

GRIMES

ENVIRO TECH

ERNST

BALLARD

SPANGLER

SITE TECH

GAMMILL

STEPP

HULMAN

PROJECT MANAGEMENT

BOYD

P. COLLINS

HOUSTON

PETERSON

MELTZ

HELTEMES

SKOVHOLT

REACTOR SAFETY

ROSS

NOVAK

ROSZTOCZY

CHECK

AT & I

SALTZMAN

RUTBERG

SITE SAFETY & ENVIRO

ANALYSIS

DENTON & MULLER

SITE ANALYSIS

VOLLMER

BUNCH

J. COLLINS

KREGER

EXTERNAL DISTRIBUTION

LPDR: Ft Pierce, Fla

TIC

NSIC

ASLB

ACRS 16 HOLDING/SENT

NATL LAB

REG. V-IE

LA PDR

CONSULTANTS

BROOKHAVEN NATL LAB

ULRIKSON (ORNL)

CONTROL NUMBER

3912



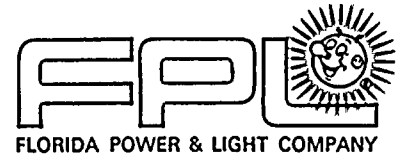
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April 16, 1976
L-76-159

Director of Nuclear Reactor Regulation
Attention: Mr. Vincent Stello, Jr., Director
Division of Operating Reactors
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555



Dear Mr. Stello:

Re: St. Lucie Unit 1
Docket No. 50-335
Proposed Amendment to Facility
Operating License DPR-67

In accordance with 10 CFR 50.30, Florida Power and Light Company submits herewith three (3) signed originals and forty (40) conformed copies of a request to amend Facility Operating License DPR-67.

The proposed change is as described below and as shown in the accompanying technical specification page bearing the date of this letter in the lower right hand corner.

This submittal proposes deleting the special test exception from Technical Specification 3.1.1.5 which would in effect prohibit the reactor from being critical below 515°F at any time.

We also propose that condition A3 to Enclosure 1 of the operating license be deleted.

These amendments are requested because difficulty has been experienced in withdrawing CEDM 44 at cold conditions. During cold rod drop time testing, it was determined that CEDM 44 was the second fastest rod. The cold rod drop time test procedure calls for the fastest and second fastest rods to be extensively tested. In lieu of this requirement, we have performed these same extensive tests on the fastest and third fastest CEDM's with satisfactory results.

We propose to drop time test CEDM 44 10 times at hot full flow conditions and in addition plan to repair the CEDM at the next extended reactor shutdown which would be expected to last at least 2 weeks.



• - - -

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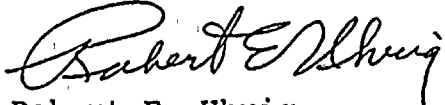
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11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 100

Director of Nuclear Reactor Regulation
Attention: Mr. Vincent Stello, Jr., Director
Page Two
April 16, 1976

The proposed amendment has been reviewed and the conclusion reached that it does not involve a significant hazards consideration, therefore, prenoticing pursuant to 10 CFR 2.105 should not be required.

Very truly yours,



Robert E. Uhrig
Vice President

GEL/cpc

Attachment

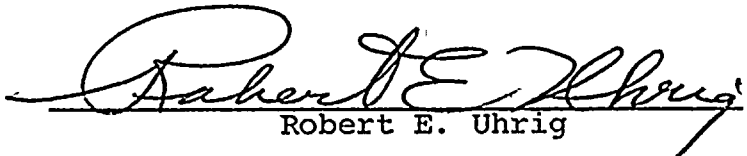
cc: Mr. Norman C. Moseley
Jack R. Newman, Esquire

STATE OF FLORIDA)
) SS
COUNTY OF DADE)

Robert E. Uhrig, being first duly sworn, deposes and says:

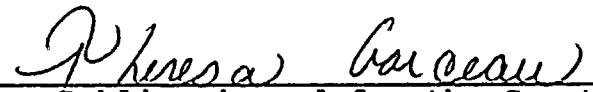
That he is a Vice President of Florida Power & Light Company,
the Licensee herein;

That he has executed the foregoing document; that the statements
made in this said document are true and correct to the best of
his knowledge, information and belief; and that he is authorized
to execute the document on behalf of said Licensee.


Robert E. Uhrig

Subscribed and sworn to before me

this 16th day of April, 1976


Notary Public, in and for the County of
Dade, State of Florida

My commission expires: _____

NOTARY PUBLIC STATE OF FLORIDA AT LARGE
MY COMMISSION EXPIRES JAN. 25, 1979
BONDED THRU GENERAL INSURANCE UNDERWRITERS

REACTIVITY CONTROL SYSTEMS

MINIMUM TEMPERATURE FOR CRITICALITY

LIMITING CONDITION FOR OPERATION

3.1.1.5 The Reactor Coolant System lowest operating loop temperature (T_{avg}) shall be $\geq 515^{\circ}\text{F}$ when the reactor is critical.

APPLICABILITY: MODES 1 and 2[#].

ACTION:

With a Reactor Coolant System operating loop temperature (T_{avg}) $< 515^{\circ}\text{F}$, restore T_{avg} to within its limit within 15 minutes or be in ^{HOT}STANDBY within the next 15 minutes.

SURVEILLANCE REQUIREMENTS

4.1.1.5 The Reactor Coolant System temperature (T_{avg}) shall be determined to be $\geq 515^{\circ}\text{F}$.

- a. Within 15 minutes prior to achieving reactor criticality, and
- b. At least once per 30 minutes when the reactor is critical and the Reactor Coolant System temperature (T_{avg}) is $< 525^{\circ}\text{F}$.

[#] With $K_{eff} \geq 1.0$.

SAFETY EVALUATION

The Regulatory Position C.1 of Regulatory Guide 1.68 discusses cold and hot rod drop testing of pressurized water reactors. The regulatory position recognizes that cold rod tests are performed to provide reasonable assurance that the hot tests will be successful.

An additional concern that would indicate the desirability of conducting cold rod drop time tests is that at some point in time, consideration may be given to the use of nuclear heat at low temperatures to expedite the heatup of the reactor coolant system.

The Technical Specifications for St. Lucie No. 1 permit criticality below 515°F under certain test conditions.

The position of Regulatory Guide 1.68 has been addressed by the successful completion of hot rod drop time tests.

Criticality below 515°F has been prohibited by removing the special test exception from the Technical Specifications.

Additional testing of the CEDM will demonstrate that the ability to effectively shutdown the reactor has not been impaired.

Therefore we have determined that the probability of occurrence or the consequences of an accident or malfunction of equipment important to safety previously evaluated in the Safety Analysis Report has not been increased; that the probability for an accident or malfunction of a different type than any evaluated previously in the Safety Analysis Report has not been created; and that the margin to safety as defined in the bases for the Technical Specifications has not been reduced.

