

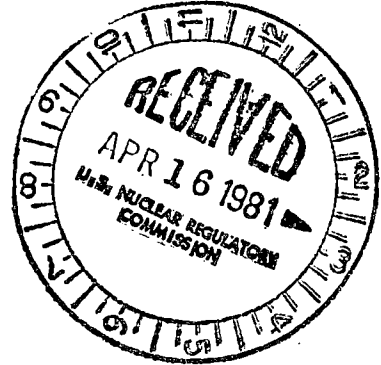


**PSEG**

Public Service Electric and Gas Company P.O. Box #168 Hancocks Bridge, New Jersey 08038

April 10, 1981

Director, Office of Inspection and Enforcement  
U.S. Nuclear Regulatory Commission  
Washington, DC 20555



Dear Sir:

MONTHLY OPERATING REPORT  
SALEM NO. 2  
DOCKET NO. 50-311

In compliance with section 6.9, Reporting Requirements for the Salem Technical Specifications, 10 copies of the following monthly operating reports for the month of March 1981 are being sent to you.

Average Daily Unit Power Level  
Operating Data Report  
Unit Shutdowns and Power Reductions  
Major Plant Modification  
Summary of Safety Related Maintenance  
Refueling Information

Sincerely yours,

H. J. Midura  
Manager - Salem Generating Station

LKM:pl

cc: Mr. Boyce H. Grier  
Director of U.S. NRC  
Office of Inspection and Enforcement  
Region I  
631 Park Avenue  
King of Prussia, PA 19406

Director, Office of Management  
Information and Program Control  
U.S. Nuclear Regulatory Commission  
Washington, DC 20555

*Handwritten:* A006  
51/1

Enclosures  
Page 1 of 10  
8-1-7.R4

The Energy People

# AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-311

UNIT Salem #2

DATE April 10, 1981

COMPLETED BY L.K. Miller

TELEPHONE 609-365-7000 X507

MONTH March 1981

## DAY AVERAGE DAILY POWER LEVEL

(MWe-NET)

1	0
2	0
3	0
4	0
5	0
6	0
7	0
8	0
9	0
10	0
11	0
12	0
13	0
14	0
15	0
16	0

## DAY AVERAGE DAILY POWER LEVEL

(MWE-NET)

17	0
18	0
19	0
20	0
21	0
22	0
23	0
24	0
25	0
26	0
27	0
28	0
29	0
30	0
31	0

OPERATING DATA REPORT

DOCKET NO.: 50-311

DATE: April 10, 1981

COMPLETED BY: L.K. Miller

TELEPHONE: 609-365-7000 X507

OPERATING STATUS

1. Unit Name: Salem #2
2. Reporting Period: March 1981
3. Licensed Thermal Power (MWt): 3411
4. Nameplate Rating (Gross MWe): 1162
5. Design Electrical Rating (Net MWe): 1115
6. Maximum Dependable Capacity (Gross MWe): 1149
7. Maximum Dependable Capacity (Net MWe): 1104
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reason:

Notes:

NONE

9. Power Level To Which Restricted, If Any (Net MWe): 5% Thermal
10. Reasons For Restrictions, If Any: Pending Full Power Operating License

	This Month	Year to Date	Cumulative
11. Hours In Reporting Period	<u>744</u>	<u>2160</u>	<u>8329</u>
12. Number Of Hours Reactor Was Critical	<u>0</u>	<u>0</u>	<u>268.2</u>
13. Reactor Reserve Shutdown Hours	<u>0</u>	<u>0</u>	<u>0</u>
14. Hours Generator On-Line	<u>0</u>	<u>0</u>	<u>0</u>
15. Unit Reserve Shutdown Hours	<u>0</u>	<u>0</u>	<u>0</u>
16. Gross Thermal Energy Generated (MWH)	<u>0</u>	<u>0</u>	<u>0</u>
17. Gross Electrical Energy Generated (MWH)	<u>0</u>	<u>0</u>	<u>0</u>
18. Net Electrical Energy Generated (MWH)	<u>0</u>	<u>0</u>	<u>0</u>
19. Unit Service Factor	<u>0</u>	<u>0</u>	<u>0</u>
20. Unit Availability Factor	<u>0</u>	<u>0</u>	<u>0</u>
21. Unit Capacity Factor (Using MDC Net)	<u>0</u>	<u>0</u>	<u>0</u>
22. Unit Capacity Factor (Using DER Net)	<u>0</u>	<u>0</u>	<u>0</u>
23. Unit Forced Outage Rate	<u>0</u>	<u>0</u>	<u>0</u>
24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):			

NONE

25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A
26. Units In Test Status (Prior to Commercial Operation):

	Forecast	Achieved
INITIAL CRITICALITY	<u>6/30/80</u>	<u>8/2/80</u>
INITIAL ELECTRICITY	<u>9/01/80</u>	<u>NA</u>
COMMERCIAL OPERATION	<u>NA</u>	<u>NA</u>

## UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH March 1981DOCKET NO.: 50-311UNIT NAME: Salem #2DATE: April 10, 1981COMPLETED BY: L.K. MillerTELEPHONE: 609-365-7000 X507

NO.	DATE	TYPE <sup>1</sup>	DURATION (HOURS)	REASON <sup>2</sup>	METHOD OF SHUTTING DOWN REACTOR	LICENSE EVENT REPORT #	SYSTEM CODE <sup>4</sup>	COMPONENT CODE <sup>5</sup>	CAUSE AND CORRECTIVE ACTION TO PREVENT RECURRENCE
									N/A

<sup>1</sup>  
F: Forced  
S: Scheduled

<sup>2</sup>  
Reason:  
A-Equipment Failure(Explain)  
B-Maintenance or Test  
C-Refueling  
D-Regulatory Restriction  
E-Operator Training & License Examination  
F-Administrative  
G-Operational Error(Explain)  
H-Other (Explain)

<sup>3</sup>  
Method:  
1-Manual  
2-Manual Scram.  
3-Automatic Scram.  
4-Continuation of  
Previous Outage  
5-Load Reduction  
9-Other

<sup>4</sup>  
Exhibit G - Instructions  
for Preparation of Data  
Entry Sheets for Licensee  
Event Report(LER) File  
(NUREG-0161)

<sup>5</sup>  
Exhibit I-Same  
Source

## MAJOR PLANT MODIFICATIONS

REPORT MONTH March 1981DOCKET NO: 50-311UNIT NAME: Salem 2DATE: April 10, 1981COMPLETED BY: L.K. MillerTELEPHONE: 609-365-7000 X507

*DCR NO.	PRINCIPLE SYSTEM	SUBJECT
2ET-0760	Auxiliary Feedwater	Perform 48 Hour Endurance Test
2ET-0917	Condensate Polishing	Install Upper Interface Sight Glass
2EC-0940	Main Generator	Weld Aluminum Plates on Terminal Box for Overheating Problem
2EC-1049	Structural	Add Structural Reinforcing in Block Walls
2ET-1120	Safeguard Equipment Control	Modify Auto Test Feature

\* DESIGN CHANGE REQUEST

8-1-7.R1

Page 5 of 10

DOCKET NO.: 50-311

UNIT NAME: Salem #2

REPORT MONTH March 1981

**DATE:** April 10, 1981

COMPLETED BY: L.K. Miller

**TELEPHONE:** 609-365-7000 X507

*DCR NO.	10CFR50.59	SAFETY EVALUATION
2ET-0760	This DCR provides for the running of the auxiliary feedwater pumps for a 48 hr. endurance test as required by the NRC. It does not involve either an unreviewed safety or environmental question.	
2EC-0917	This change involves the installation of an additional sight glass in the condensate polishing system cation regeneration vessel. This modification does not involve an unreviewed safety question. The installation does not constitute a new hazard not previously analyzed.	
2EC-0940	This is a non-safety related system and as such the modification will not affect safety margin or have any safety implications.	
2EC-1049	Issuance of the DCR provides structural reinforcing to existing block wall in order to protect safety related equipment in the event of an earth quake. Installation will not affect safe shut down of NSSS equipment or building integrity adversely.	
2ET-1120	This testing DCR improves reliability of a circuit test function, circuit function is not affected. Does not involve any unreviewed safety or environmental question.	

Page 6 of 10

SALEM GENERATING STATION  
SAFETY RELATED EQUIPMENT WORK ORDER LOG  
UNIT 2

MAR 31 1981

WORK ORDER NUMBER	DEPT	EQUIPMENT IDENTIFICATION	EXPLANATION OF WORK PERFORMED
919907	MD	2A SEC Alarm	Description of Problem - Trouble alarm will not clear Corrective Action Taken - Vendor did modification on card file
920509	MD	Valves, 2CS12 & 13	Description of Problem - Leaking Corrective Action Taken - Replaced valves
939636	MD	Pump, No. 24 Service Water	Description of Problem - Motor upper bearing temperature too high Corrective Action Taken - Oil was changed
939643	MD	Heat Trace Circuit HC-2109	Description of Problem - Temperature indicators are below specs Corrective Action Taken - Replaced primary and secondary heat tapes
942276	MD	Valve, 2NT5	Description of Problem - Valve inoperable due to broken handwheel Corrective Action Taken - Replaced valve
942278	MD	Valve, 2NT26	Description of Problem - Failed leak rate test Corrective Action Taken - Applied new disc
942290	MD	Hanger Support 2-RHRH-22-34	Description of Problem - East snubber failed manual stroke test Corrective Action Taken - Installed new snubber
942478	MD	Valve, 21SJ44	Description of Problem - Leakage Corrective Action Taken - Repaired stem limit switch
942512	MD	Vital Heat Trace - ECCS	Description of Problem - Love controller 2633A, wire #81 indicates zero amps with contactor closed Corrective Action Taken - Replaced heat tape
942513	MD	Vital Heat Trace - ECCS	Description of Problem - Love controller 2610B, wire #17 indicates zero amps with contactor closed Corrective Action Taken - Replaced heat tape

MAR 31 1981

WORK ORDER NUMBER	DEPT	EQUIPMENT IDENTIFICATION	EXPLANATION OF WORK PERFORMED
942617	MD	2A SEC Alarm Cabinet	Description of Problem - Alarm will not clear Corrective Action Taken - Changed chassis
942720	MD	Valve, 2LSJ134	Description of Problem - Valve will not close from Control Room Corrective Action Taken - Replaced torque switch
942722	MD	Valve, 2SJ135	Description of Problem - Will not close from Control Room Corrective Action Taken - Valve blocked open electrically
944392	MD	#21 Boric Acid Transfer Pump	Description of Problem - Heater element is not operating Corrective Action Taken - Reconnected wiring and installed insulation
944393	MD	#22 Boric Acid Transfer Pump	Description of Problem - Heater element is energized at all times Corrective Action Taken - Thermostats tagged off
944435	MD	#21 Boric Acid Transfer Pump	Description of Problem - Strip heaters are burned out Corrective Action Taken - Reconnected wiring and installed insulation
947562	MD	Valve, 2CS17	Description of Problem - Not operating correctly Corrective Action Taken - Replaced female disc
906609	PD	Rad Monitoring	Description of Problem - Repair portable connection box #39 connector in containment Corrective Action Taken - Replaced connector
910671	PD	Valve, 2SW169	Description of Problem - Solenoid valve appears to be sticking Corrective Action Taken - Stroked valve
928261	PD	Channel 2TM-4125	Description of Problem - Out of calibration Corrective Action Taken - Replaced 2 capacitors C23 and C24
928262	PD	No. 24 Steam Generator Steam Dump, 2PT-546B	Description of Problem - Channel out of calibration Corrective Action Taken - Removed controller 40-149 and installed 238



WORK ORDER NUMBER	DEPT	EQUIPMENT IDENTIFICATION	EXPLANATION OF WORK PERFORMED
932593	PD	Valve, 22CV160	Description of Problem - Operation is sluggish Corrective Action Taken - Replaced servo amp
939612	PD	Pump, 21 Charging	Description of Problem - Gear drive is leaking oil out of local temperature indicator Corrective Action Taken - Tightened temperature well
939646	PD	Valve, 21CV160	Description of Problem - Limit switch not making up Corrective Action Taken - Adjusted limit switch
942298	PD	Valve, 22MS10	Description of Problem - Leaking diaphragm Corrective Action Taken - Replaced diaphragm
942712	PD	Monitor, 2R5	Description of Problem - Monitor is alarming with very low reading Corrective Action Taken - Changed micro processor (MC-6800) and reprogrammed 2R5
942866	PD	RMS 2R17A/2R17B	Description of Problem - Fail/ackn flashing locally. Unable to clear "no scan" on CRT Corrective Action Taken - Replaced damaged pin on back plane of 2R17A and cleared RAMS on 2R17B
942879	PD	Valve, 22SW57	Description of Problem - Fails to open Corrective Action Taken - Recalibrated transmitter (PA1634Z)
942927	PD	Valve, 21SW49	Description of Problem - Valve will not respond to increase in header pressure in auto mode Corrective Action Taken - Set controller to control at 130 psi
944515	PD	Valve, 2VC11	Description of Problem - Failed leak rate test Corrective Action Taken - Reworked operator and limit switches

MAR 31 1981

# REFUELING INFORMATION

DOCKET NO.: 50-311

UNIT: Salem #2

DATE: April 10, 1981

COMPLETED BY: L.K. Miller

TELEPHONE: 609-365-7000 X507

MONTH: March 1981

1. Refueling information has changed from last month:

YES \_\_\_\_\_ NO X

2. Scheduled date of next refueling: December 4, 1982

3. Scheduled date for restart following refueling: February 27, 1983

4. A. Will Technical Specification changes or other license amendments be required? YES \_\_\_\_\_ NO \_\_\_\_\_

NOT DETERMINED TO-DATE February 1981

B. Has the reload fuel design been reviewed by the Station Operating Review Committee? YES \_\_\_\_\_ NO X

If no, when is it scheduled? November 1982

5. Scheduled date(s) for submitting proposed licensing action:

November 1982 (If Required)

6. Important licensing considerations associated with refueling:

NONE

7. Number of Fuel Assemblies:

A. In-Core 193

B. In Spent Fuel Storage 0

8. Present licensed spent fuel storage capacity: 1,170

Future spent fuel storage capacity: 1,170

9. Date of last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity: March 2000