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 FACIL: 50-220 Nine Mile Point Nuclear Station, Unit 1, Niagara Power 050002
 AUTH: NAME AUTHOR AFFILIATION 50-220
 DISE: D.P. Niagara Mohawk Power Corp.
 RECIP: NAME RECIPIENT AFFILIATION
 IPPOLITO, T.A. Operating Reactors Branch 3

SUBJECT: Forwards info re containment isolation barriers in response
 to NRC 790713 ltr. Fourteen oversize drawings, available in
 Central Files only, encl. *See Reports*

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*"Report Jacket, Information Re Containment
 Isolation Barriers."
 (DRWGS ENCLOSED)*

DEC 6 1979
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Donald P. Dise
Vice President
Engineering

NIAGARA MOHAWK POWER CORPORATION/300 ERIE BOULEVARD WEST, SYRACUSE, N.Y. 13202/TELEPHONE (315) 474-1511

November 28, 1979

Director of Reactor Regulation
Attn: Mr. Thomas Ippolito, Chief
Operating Reactors Branch #3
U. S. Nuclear Regulatory Commission
Washington, D.C. 20555

Dear Mr. Ippolito:

Re: Nine Mile Point Unit 1
Docket No. 50-220
DPR-63

Your letter dated July 13, 1979 requested information regarding the containment isolation barriers. The attached responds to your request.

Very truly yours,

NIAGARA MOHAWK POWER CORPORATION

A handwritten signature in cursive script, appearing to read 'Donald P. Dise'.

Donald P. Dise
Vice President Engineering

PEF:jk

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DRAWINGS to f/ES
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PLANT Nine Mile Point Unit 1

BYPASS CAPACITY

Plant Steam Bypass Capacity 40% rated steam flow

50-220
Ltr 11-28-79
7912040 469

RETURN TO REACTOR DOCKET FILES

1 K 2 3 4 5 6 7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 100

1 2 3 4 5 6 7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 100

PLANT Nine Mile Point Unit 1

SYSTEMS AND COMPONENTS SHARED BETWEEN UNITS

Single-unit plant check here ☒ and do not complete

System or Component

Shared Between
Units Numbers

N/A

N/A

PLANT

Nine Mile Point Unit 1

PLANT-SPECIFIC SYSTEM INFORMATION

<u>System</u>	<u>General</u>		<u>Water Sources</u>		<u>Instrumentation and Control.</u>		<u>Frequency of System and Component Tests</u>
	<u>Safety Classification</u>	<u>Seismic Category</u>	<u>Safety Classification</u>	<u>Seismic Category</u>	<u>Safety Classif.</u>	<u>Seismic Category</u>	
Emergency Condenser	Safety Related	I	Safety Related	I	Safety Rel.	I	Once each operating cycle
High Pressure Coolant Injection	Safety Related	I	Safety Related	I	Safety Rel.	I	Pump operability is demonstrated once per quarter. Automatic startup of system is demonstrated once per operating cycle
Low Pressure Core Spray	Safety Related	I	Safety Related	I	Safety Rel.	I	Pumps and power operated valves are checked once per quarter. Automatic startup of one set of pumps is demonstrated once each operating cycle
Auto Depressurization System	Safety Related	I	N/A	N/A	Safety Rel.	I	Automatic initiation is demonstrated once per operating cycle
Pressure Relief Valves	Safety Related	I	N/A	N/A	N/A	N/A	Automatic initiation is demonstrated once per operating cycle.

PLANT Nine Mile Point Unit 1

PLANT-SPECIFIC SYSTEM INFORMATION

<u>System</u>	<u>General</u>		<u>Water Sources</u>		<u>Instrumentation and Control</u>		<u>Frequency of System and Component Tests</u>
	<u>Safety Classification</u>	<u>Seismic Category</u>	<u>Safety Classification</u>	<u>Seismic Category</u>	<u>Safety Classif.</u>	<u>Seismic Category</u>	
Safety Valves	Safety Related	I	N/A	N/A	N/A	N/A	Eight of the 16 safety valves are removed, tested for setpoint and partial lift once per operating cycle
Shutdown Cooling System	Safety Related	I	Safety Related	I	Safety Rel.	I	Pump operability is checked once per quarter
Containment Spray and Suppression Pool Cooling System	Safety Related	I	Safety Related	I	Safety Rel.	I	Pump operability is checked once per quarter. Automatic start-up of containment spray pumps is demonstrated once per operating cycle
Emergency Service Water	Safety Related	I	Safety Related	I	Safety Rel.	I	Pumps operability is checked once per quarter.

PLANT Nine Mile Point Unit 1

PLANT-SPECIFIC SYSTEM INFORMATION

<u>System</u>	<u>General</u>		<u>Water Sources</u>		<u>Instrumentation and Control</u>		<u>Frequency of System and Component Tests</u>
	<u>Safety Classification</u>	<u>Seismic Category</u>	<u>Safety Classification</u>	<u>Seismic Category</u>	<u>Safety Classif.</u>	<u>Seismic Category</u>	
Reactor Building Closed Loop Cooling Water System	Safety Related	I	Safety Related	I	Safety Rel.	I	Normally in service
Control Rod Drive	Safety Related	I	Safety Related	I	Safety Rel.	I	Automatic startup of each pump is demonstrated once per operating cycle
Condensate Storage Tank	Safety Related	I	Non Safety Related	Non Seismic	Non Safety Related	Non Seismic	N/A
Main Feedwater (pressure boundary)	Safety Related	I	Safety Related	I	Non Safety Related	Non Seismic	Normally in service
Cooling Water to Recirculation Pump/Motor	Safety Related	I	Safety Related	I	Safety Rel.	I	Normally in service

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Prim. Cont. Penetration Number	Line Size, In.	System	Is System an engineered safety function	Figure	Process Fluid	Valve Number	Isolation Signal Code(s)	Location	Type	Actuator	Primary Actuation Mode	Secondary Actuation Mode	Full Closure Time, sec.	Power Source	Position Indication in Control Rm.	Positions				Comments
																Normal	Shutdown	Post Accident	Power Failure	
X-12B	8	RBCLC to Drywell Coolers	N	1	W	70-95	N/A	0	CK	N/A	RF	N/A	N/A	N/A	N/A	N/A	N/A	N/A	N/A	
X-13B	8	RBCLC from Dry. Clrs	N	1	W	70-94	Manual	0	GT	MO	RMC	N/A	30	DC	D	0	0	0	AI	
X-13A	12	Core Spray Inlet	Y	1	W	40-01	Manual	I	GT	MO	A	RMC	90	AC	D	C	C	0(3)	AI	
X-13A	12	Core Spray Inlet	Y	1	W	40-09	Manual	I	GT	MO	A	RMC	90	AC	D	C	C	0(3)	AI	
X-13A	12	Core Spray Inlet	Y	1	W	40-02	Manual	0	GT	MO	A	RMC	N/A	AC	D	LO	0	0(3)	AI	
X-14	12	Core Spray Inlet	Y	1	W	40-10	Manual	I	GT	MO	A	RMC	90	AC	D	C	C	0(3)	AI	
X-14	12	Core Spray Inlet	Y	1	W	40-11	Manual	I	GT	MO	A	RMC	90	AC	D	C	C	0(3)	AI	
X-14	12	Core Spray Inlet	Y	1	W	40-12	Manual	0	GT	MO	A	RMC	N/A	AC	D	LO	0	0(3)	AI	
X-18	24	Drywell Vent & Purge	N	2	A	201-09	E	0	B	MO	A	RMC	60	AC	D	C	0	C	AI	
X-18	24	Air Supply	N	2	A	201-10	E	0	B	AO	A	RMC	60	A	D	C	0	C	C	
X-19	20	Drywell N2 Vent&Fill	N	2	N2	201-31	E	0	B	MO	A	RMC	60	AC	D	C	0	C	AI	
X-19	20	Drywell N2 Vent&Fill	N	2	N2	201-32	E	0	B	AO	A	RMC	60	A	D	C	0	C	C	
X-19	4	Drywell N2 Vent&Fill	N	2	N2	201.2-03	E	0	DCV	AO	A	RMC	60	A	D	0	0	C	C	
X-19	4	Drywell N2 Vent&Fill	N	2	N2	201.2-32	E	0	DCV	AO	A	RMC	60	A	D	0	0	C	C	
X-20	1	Cont. Atmos. Dil. Sample Ret.	N	2	A	201.7-10	E	0	DCV	AO	A	RMC	60	A	D	0	0	C	C	
X-20	1	Cont. Atmos. Dil. Sample Ret.	N	2	A	201.7-11	E	0	DCV	AO	A	RMC	60	A	D	0	0	C	C	
X-25	4	Drywell Floor Dr. Sump Outlet	N	1	W	83.1-11	E	I	GT	MO	A	RMC	60	AC	D	0	0	C	AI	
X-25	4	Drywell Floor Dr. Sump Outlet	N	1	W	83.1-12	E	0	DCV	AO	A	RMC	60	A	D	0	0	C	C	



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PRIMARY CONTAINMENT ISOLATION SYSTEM DATA
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Isolation Valves

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Isolation Valves

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Isolation Valves

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Isolation Valves

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PRIMARY CONTAINMENT ISOLATION SYSTEM DATA
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Isolation Valves

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PRIMARY CONTAINMENT ISOLATION SYSTEM DATA
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Isolation Valves

Prim. Cont. Penetration Number	Line Size, In.	System	Is System an Engineered safety function	Figure	Process Fluid	Valve Number	Isolation Signal Code(s)	Location	Type	Actuator	Primary Actua- tion Mode	Secondary Actua- tion Mode	Full Closure Time, sec.	Power Source	Position Indica- tion in Control Rm.	Positions				Comments
																Normal	Shutdown	Post Accident	Power Failure	
X-139	1	Reactor Water Sample	N	6	W		N/A	0	FF	N/A	HF	N/A	N/A	N/A	N/A	N/A	N/A	N/A	N/A	
X-139	1	Recirculation Sys.	N	6	W		N/A	0	GB	N/A	M	N/A	N/A	H	N	C	C	C	N/A	
X-140	12	Cont. Spray Inlet	Y	1	W	80-17	N/A	0	CK	N/A	N/A	N/A	N/A	N/A	N/A	N/A	N/A	N/A	N/A	
X-140	12	Cont. Spray Inlet	Y	1	W	80-15	Manual (6)	0	GT	AO	RMC	-	60	A	D	0	0	0	0	
X-149	12	Cont. Spray Inlet	Y	1	W	80-38	N/A	0	CK	N/A	RF	N/A	N/A	N/A	N/A	N/A	N/A	N/A	N/A	
X-149	12	Cont. Spray Inlet	Y	1	W	80-36	Manual (6)	0	GT	AO	RMC	-	60	A	D	0	0	0	0	
X-150	12	Cont. Spray Inlet	Y	1	W	80-37	N/A	0	CK	N/A	RF	N/A	N/A	N/A	N/A	N/A	N/A	N/A	N/A	
X-150	12	Cont. Spray Inlet	Y	1	W	80-35	Manual (6)	0	GT	AO	RMC	-	60	A	D	0	0	0	0	
X-156	4	Recirc. Pump Cooling	N	1	W		N/A	I	GB	N/A	M	N/A	N/A	H	N	0	0	0	N/A	
X-156	4	Water Return	N	1	W	70-92	Manual	0	GB	MO	RMC	N/A	30	DC	D	0	0	C	AI	
X-157	4	Recirculation Pump	N	1	W		Manual	I	GT	SO	RM	N/A	N/A	DC	D	0	0	0	N/A	
X-157	4	Cooling Water Sup.	N	1	W	70-93	N/A	0	CK	N/A	RF	N/A	N/A	N/A	N/A	N/A	N/A	N/A	N/A	
X-168	3/4	Dry Level Imp. Line	N	2	A		Manual	0	GT	N/A	M	N/A	N/A	H	N	0	0	0	N/A	
X-174	3	Control Rod Drive to Reactor	N	1	W	301- 114	N/A	I	GT	N/A	M	N/A	N/A	H	LO	0	0	0	N/A	
X-174	3	Control Rod Drive to Reactor	N	1	W	301- 113	N/A	I	CK	N/A	RF	N/A	N/A	N/A	N/A	N/A	N/A	N/A	N/A	
X-174	3	Control Rod Drive to Reactor	N	1	W	301- 112	N/A	0	CK	N/A	RF	N/A	N/A	N/A	N/A	N/A	N/A	N/A	N/A	
X-230	3/4	N ₂ Purge to TIP	N	1	N ₂	201.2- 65	N/A	0	GT	N/A	M	N/A	N/A	H	LO	0	0	0	N/A	
X-230	3/4	N ₂ Purge to TIP	N	1	N ₂	201.2- 39	N/A	0	CK	N/A	RF	N/A	N/A	N/A	N/A	N/A	N/A	N/A	N/A	
X-230	3/4	N ₂ Purge to TIP	N	1	N ₂	201.2- 40	N/A	0	CK	N/A	RF	N/A	N/A	N/A	N/A	N/A	N/A	N/A	N/A	
X-121	3/4	Breathing Air	N	8	A		N/A	I	GT	N/A	M	N/A	N/A	H	N	LC	0	LC	N/A	
X-121	3/4	For Drywell	N	8	A		N/A	0	GT	N/A	M	N/A	N/A	H	N	C	0	C	N/A	
X-121	3/4	" " "	N	8	A		N/A	0	GT	N/A	M	N/A	N/A	H	N	C	0	C	N/A	



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PRIMARY CONTAINMENT ISOLATION SYSTEM DATA
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Isolation Valves

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PRIMARY CONTAINMENT ISOLATION SYSTEM WATER

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Footnotes for Isolation Valve Table

1. The valves in the emergency condenser steam supply line remain open during an accident unless there is a break in the emergency condenser line, indicated by high steam flow in the emergency condenser line or high radiation in the emergency condenser vents. These signals automatically close the valves.
2. The air operated valve in the emergency condenser remains closed during accident conditions. They are opened by high reactor pressure or low-low water level signals. The air operated valves will then remain opened unless a break in an emergency condenser line is indicated as discussed in (1) above.
3. The Core Spray System is considered to be an extension of containment, therefore core spray valves 40-01, 40-09, 40-10 and 40-11 do not automatically isolate. These valves open on a low reactor pressure signal in conjunction with a high drywell pressure or low-low reactor water level. If the Core Spray System is not needed to maintain reactor vessel water level, these valves can be isolated manually.
4. The drywell vent and fill line consists of a 20-inch line which penetrates primary containment. Once outside primary containment, it branches into a 24-inch vent line and a 4-inch nitrogen supply line with each line containing two isolation valves.
5. There are two lines per penetration for the reactor recirculation system instrumentation. Each line has a manual gate valve and a flow check valve outside containment.
6. The Containment Spray System is considered to be an extension of containment, therefore, the containment spray valves do not automatically isolate. If the Containment Spray System is not required to mitigate the consequences of an accident, it can be manually isolated from the control room.

NINE MILE POINT UNIT 1
PRIMARY CONTAINMENT ISOLATION SYSTEM DATA

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ABBREVIATIONS

Engineered Safety Function

N = NO
Y = YES

Position Indication in Control Room

D = Direct
I = Indirect
N = None
Others stated in Table

Fluid

A = Air
B = Sodium Pentaborate
S = Steam
W = Water
N₂ = Nitrogen

Isolation Valve Location

I = Inside Containment
O = Outside Containment
Others stated in Table

Isolation Valve Actuation Mode

A = Automatic
HF = High Flow
M = Manual
OP = Overpressure
RF = Reverse Flow
RMC = Remote Manual Control Room
RM = Remote Manual (Local)

Isolation Valve Positions

AI = As Is
C = Closed
O = Open

Isolation Valve Type

A = Angle
B = Butterfly
BCK = Ball Check
BL = Ball
CK = Check
DCV = Diaphragm
Control Valve
FCV = Flow Check Valve
FF = Flow Fuse
GB = Globe
GT = Gate
RV = Relief
SCV = Stop Check
SV = Solenoid
VB = Vacuum Breaker

Isolation Valve Power Source

A = Air
AC = AC
DC = DC
H = Hand

Isolation Valve Actuator

AO = Air
MO = Motor
SO = Solenoid

Isolation Signal Codes

Code or Group	Parameter(s) Sensed for Isolation	Set Point (units)
A	High Steam Flow - Main Steam Line	≤ 105 psid
	High Radiation - Main Steam Line	≤ 5 times normal background
	Low Reactor Pressure	≥ 850 psig
	Low Low Low Condenser Vacuum	≥ 7 inches mercury vacuum
B	High Temperature - Main Steam Line Tunnel	≤ 200F
	Low Low Reactor Water Level	≥ 5 inches Indicator Scale
	High Steam Flow - Emergency Cooling System	≤ 19 psid
	High Radiation - Emergency Cooling System Vent	≤ 25 mr/hr

NINE MILE POINT UNIT 1
PRIMARY CONTAINMENT ISOLATION SYSTEM DATA

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ABBREVIATIONS (Continued)

Isolation Signal Codes
(Continued)

<u>Code or Group</u>	<u>Parameter(s) Sensed for Isolation</u>	<u>Set Point (units)</u>
C	Manual	N/A
D	Low-Low Reactor Water Level	> 5 inches (Indicator Scale)
	High Area Temperature	≤ 190F for Cleanup System ≤ 170F for Shutdown Cooling System
E	Low-Low Reactor Water Level	> 5 inches (Indicator Scale)
	High Drywell Pressure	≤ 3.5 psig

NINE MILE POINT UNIT 1
PRIMARY CONTAINMENT ISOLATION SYSTEM DATA

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Figure Codes

- 1 = C-18006-C
- 2 = C-18014-C
- 3 = C-18015-C
- 4 = C-18017-C
- 5 = C-18020-C
- 6 = C-18041-C
- 7 = C-18022-C
- 8 = C-18578-C
- 9 = C-18012-C
- 10 = C-18002-C

PLANT Nine Mile Point Unit 1

DESIGN REQUIREMENTS FOR CONTAINMENT ISOLATION BARRIERS

Question: Discuss the extent to which the quality standards and seismic design classification of the containment isolation provisions follow the recommendations of Regulatory Guides 1.26, "Quality Group Classifications and Standards for Water-, Steam-, and Radioactive-Water-Containing Components of Nuclear Power Plants", and 1.29, "Seismic Design Classification".

Response: The codes used in the design of Nine Mile Point Unit 1 Class I systems at the time of construction were ASME Section I ANSI-B31.1-1955 and ANSI B16.5-1955 with requirements of ASME Section III for non-destructive testing. For subsequent modifications which involve the addition of a complete new system, Niagara Mohawk is committed to follow the recommendations of Regulatory Guide 1.26, Revision 3. For example, the Containment Atmospheric Dilution System, including portions of the Vent and Purge and Containment Radiation Monitoring Systems, was added subsequent to commercial operation and followed the quality standards recommended in Regulatory Guide 1.26.

The containment isolation valves are all part of Class I systems. The classifications used at the time of their design were equivalent to the seismic design classifications of Regulatory Guide 1.29.

PLANT Nine Mile Point Unit 1

PROVISIONS FOR TESTING

Question: Discuss the design provisions for testing the operability of the isolation valves.

Response: All power-operated reactor coolant system isolation valves and containment isolation valves which are normally opened can be tested for operability during power operating conditions. The Nine Mile Point Unit 1 Technical Specifications requires all normally open power-operated isolation valves (except the feedwater and main steam line power-operated isolation valves which are exercised by partial closure and subsequent re-opening at least twice a week) to be fully closed and re-opened at least once per quarter.

All power-operated isolation valves which are normally closed can be tested for operability once per operating cycle. This is considered sufficient for these valves since they are only opened during infrequent intervals.

PLANT Nine Mile Point Unit 1

CODES, STANDARDS, AND GUIDES

Question:

Identify the codes, standards, and guides applied in the design of the containment isolation system and system components.

Response:

The ASME Section I and ANSI-B16.5-1955 codes were applied to the design of isolation valves. In addition to these code requirements, ultrasonic and radiographic examinations of isolation valves were performed, as well as seismic and thermal analyses. An independent third party review of calculations was performed and found acceptable.

The Containment Isolation System electrical equipment was designed to withstand the accident environmental conditions appropriate to its location. For the drywell, these conditions were short term accident conditions of saturated steam at 62 psig and 310°F. The Reactor Building environmental accident conditions for which the Containment Isolation System electrical equipment was designed are 100 percent humidity, 150°F and a 3125 psig.

All electrical specifications for Class I electrical equipment called for seismic design analyses of either of the following:

1. The maximum ground motion acceleration is 11 percent of gravity and the maximum resulting response acceleration is 45 percent of gravity for oscillators in the period range of 0.2 to 0.3 seconds.

or

2. A minimum factor of 0.20g horizontal and 0.10g vertical should be used unless a specific dynamic analysis is made in which case it should be referred to the Purchaser for his approval.

PLANT Nine Mile Point Unit 1

NORMAL OPERATING MODES AND ISOLATION MODES

Question:

Discuss the normal operating modes and containment isolation provision and procedures for lines that transfer potentially radioactive fluids out of the containment.

Response:

All systems which could transfer potentially radioactive fluids out of primary containment would automatically isolate on a primary containment and/or reactor vessel isolation signals. The only isolation systems which do not automatically isolate on these signals are the Emergency Condenser, Containment Spray and Core Spray Systems. These systems are considered essential systems to mitigate an accident and, therefore, are not isolated. The Emergency Condenser System will isolate on system high flow or high radiation of the Emergency Condenser vents.

There are six potential pathways for radioactive gases or liquids to be transferred out of the primary containment.

- a. N₂ vent and purge system.
- b. CAD System.
- c. Drywell floor and equipment drains.
- d. Recirculation sample line to reactor building sample sink.
- e. Suppression chamber transfer to waste disposal system.

All of the above except items d and e would isolate on a containment isolation signal. Overrides are provided for items a and b such that they can be manually re-opened for controlled venting and monitoring purposes. Venting would take place through the Reactor Building Emergency Ventilation System. This system would not isolate on high radiation. By procedure, venting is allowed only after containment atmosphere has been sampled and analyzed.

The drywell floor and equipment drains transfer liquid under normal operation. These lines isolate on high drywell pressure or reactor low-low water level. Since water level below the top of the fuel is required to produce significant fuel failures, highly radioactive liquid would not be automatically transferred to the waste building. Activity in these lines is not normally monitored, however, positive valve position indication is provided in the control room.

The drywell high pressure signal which initiates containment isolation has a seal-in feature so that both Reactor Protection System channels must be cleared and manual resetting accomplished before any isolation valves not provided with overrides can be re-opened. The isolation valves may then be manually opened from the control room. Thus, the pumping of drywell drains cannot be performed inadvertently.

Normal Operating Modes and Isolation Modes (Continued)

Response: The recirculation sample line contains three (3) one (1) inch manual valves in a line connected to a reactor recirculation line. These valves are normally closed except during a sampling procedure. The discharge of these 1 inch valves reduces to 1/4 inch tubing which runs around 50 feet to a sample sink. A flow fuse is installed outside of primary containment to limit flow through the line. Niagara Mohawk will install two automatic isolation valves during the next scheduled refueling outage scheduled for early 1981. These valves will close on high drywell pressure or low-low reactor water level. The isolation valves will be provided with manual overrides to permit sampling during an isolated condition.

In the interim, operating procedures will be modified to ensure that positive administrative control assure that the sample line is not inadvertently left open after use. During sampling, a member of the Operating Staff will continuously monitor sampling activities and verify that at least two valves are closed when sampling is complete.

The suppression chamber transfer to waste disposal system line requires a locked closed valve to be open to pump water out of the suppression chamber. The valve is normally locked closed except when pumping water out of the suppression chamber to maintain level.

Valves capable of automatic isolation on either low-low water level or high drywell pressure will be installed by the next refueling outage. In the interim, whenever the suppression chamber is being pumped down, a member of the operating staff will be stationed to close the valve should an isolation signal be received.

