

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

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REPORT SOURCE

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DOCKET NUMBER

EVENT DATE

REPORT DATE

### EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0	2	SEE ATTACHMENT
0	3	
0	4	
0	5	
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0	7	
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7 8 9 80

0 9		SYSTEM CODE Z Z		11	CAUSE CODE A		12	CAUSE SUBCODE B		13	COMPONENT CODE Z Z Z Z Z Z					14	COMP. SUBCODE Z		15	VALVE SUBCODE Z		16	
7	8	9	10		11		12	13		14	15	16	17	18	19	20							
17		LER/RO REPORT NUMBER		EVENT YEAR 7 9		21	22	SEQUENTIAL REPORT NO. 0 1 3		24	25	26	OCCURRENCE CODE 0 3		28	29	REPORT TYPE L		30	REVISION NO. 1		32	
ACTION TAKEN X		18	FUTURE ACTION X		19	EFFECT ON PLANT Z		20	SHUTDOWN METHOD Z		21	HOURS 0 0 0		22	ATTACHMENT SUBMITTED N		23	NPRD-4 FORM SUB. N		24	PRIME COMP. SUPPLIER Z		25
33		34	35		36	37		38	39	40	41		42		43		44		45		46		47
CAUSE DESCRIPTION AND CORRECTIVE ACTIONS																							27

### CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1	0	SEE ATTACHMENT
1	1	
1	2	
1	3	
1	4	

FACILITY STATUS (28) H (1 5) 7 8 9  
 % POWER (29) 0 0 0 10 12 13  
 OTHER STATUS (30) NA 44  
 METHOD OF DISCOVERY (31) Z 45  
 DISCOVERY DESCRIPTION (32) NA 46 80  
 ACTIVITY, CONTENT RELEASED OF RELEASE (33) Z (34) NA 44  
 AMOUNT OF ACTIVITY (35) NA 45 80  
 LOCATION OF RELEASE (36) NA 80  
 PERSONNEL EXPOSURES NUMBER (37) 0 0 0 (38) Z (39) NA 44  
 TYPE (38) Z 45  
 DESCRIPTION (39) NA 80  
 PERSONNEL INJURIES NUMBER (40) 0 0 0 (41) NA 44  
 TYPE (41) NA 45  
 DESCRIPTION (42) NA 80  
 LOSS OF OR DAMAGE TO FACILITY TYPE (43) Z (44) NA 44  
 DESCRIPTION (44) NA 45 80  
 PUBLICITY ISSUED (45) N (46) NA 44  
 DESCRIPTION (46) NA 45 80  
 NRC USE ONLY 68 69 70 71 72 73 74 75 76 77 78 79 80

PHONE: (315) 343-2110 X 1306

Event Description and Probable Consequences

During refuel outage, three Instrument Surveillance Procedures, N1-ISP-25.3, Containment Spray/Raw Water Heat Exchanger Leak Rate Test, N1-ISP-24.2, Type B Leak Rate Test Drywell Mechanical Per Double Gasket, N1-ISP-25.2, Primary Containment Isolation Leak Rate Tests, were implemented before the required temporary procedure-change signatures were obtained. Also, N1-ISP-25.3 and N1-ISP-24.2 were not reviewed by the Site Operations Review Committee within the required seven days. Since no violations of approved Operating Procedures were involved and knowledgeable Senior Reactor Operators were consulted in each case, there were minimal safety implications as a result of these instances.

Cause Description and Corrective Actions

Review of the temporary changes indicated in this report has been made by authorized personnel and subsequent approvals have been obtained from supervisors who should have documented the approvals at the time of implementation. Regular procedure changes have been reviewed by the Site Operations Review Committee and approved by the General Superintendent Nuclear Generation.