

Dominion Nuclear Connecticut, Inc.  
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dom.com



JAN 24 2018

U. S. Nuclear Regulatory Commission  
Attention: Document Control Desk  
Washington, DC 20555

Serial No. 18-012  
NSSL/MLC R0  
Docket No. 50-423  
License No. NPF-49

**DOMINION NUCLEAR CONNECTICUT, INC.**  
**MILLSTONE POWER STATION UNIT 3**  
**INSERVICE INSPECTION PROGRAM – OWNER’S ACTIVITY REPORT,**  
**REFUELING OUTAGE 18**

Dominion Nuclear Connecticut, Inc. (DNC) hereby submits the American Society of Mechanical Engineers (ASME), Section XI, Form OAR-1, Owner’s Activity Report, for the period from May 13, 2016 through Refueling Outage 18, completed on November 14, 2017 for Millstone Power Station Unit 3. The enclosure is in accordance with the requirements of ASME Code Case N-532-4.

If you have any questions or require additional information, please contact Jeffry A. Langan at (860) 444-5544.

Sincerely,

D. C. Lawrence  
Director, Nuclear Safety and Licensing – Millstone

Enclosure:

1. Owner’s Activity Report, Refueling Outage 18, Revision 0.

Commitments made in this letter: None

A047  
NRR

cc: U. S. Nuclear Regulatory Commission  
Region I  
2100 Renaissance Blvd, Suite 100  
King of Prussia, PA 19406-2713

R. V. Guzman  
Senior Project Manager – Millstone Power Station  
U. S. Nuclear Regulatory Commission  
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Rockville, MD 20852-2738

NRC Senior Resident Inspector  
Millstone Power Station

**ENCLOSURE 1**

**OWNER'S ACTIVITY REPORT**

**REFUELING OUTAGE 18**

**REVISION 0**

**DOMINION NUCLEAR CONNECTICUT, INC.  
MILLSTONE POWER STATION UNIT 3**

# MILLSTONE POWER STATION

## UNIT NO. 3

### OWNER'S ACTIVITY REPORT

### REFUELING OUTAGE 18

Revision 0

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#### Contents:

OAR-1 Report Number: MP3-3R18

Table 1: Items with Flaws or Relevant Conditions That Required Evaluation for Continued Service.

Table 2: Abstract of Repairs/Replacement Activities Required for Continued Service

Prepared By: *Mike H. Zick*  
ISI Program Owner

Date: 12/11/2017

Reviewed By: *Joel A. Zofman*  
Independent Review

Date: 12/12/17

Reviewed By: *Elizabeth K. HSI+IG*  
Authorized Nuclear Inservice Inspection

Date: 01-08-2018



# Form OAR-1 Owner's Activity Report

Attachment 1, ER-AA-ISI-100

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Report Number: MP3-3R18

Plant Millstone Power Station, Rope Ferry Road, Waterford, Connecticut 06385

Unit No. 3 Commercial service date April 26, 1986 Refueling outage no. 18  
(if applicable)

Current inspection interval 3rd  
(1<sup>st</sup>, 2<sup>nd</sup>, 3<sup>rd</sup>, 4<sup>th</sup>, other)

Current inspection period 3rd  
(1<sup>st</sup>, 2<sup>nd</sup>, 3<sup>rd</sup>)

Edition and Addenda of Section XI applicable to the inspection plans 2004 Edition, No Addenda

Date and revision of inspection plans 07/31/2017 Revision 2, Change 04-007

Edition and Addenda of Section XI applicable to repair/replacement activities, if different than the inspection plans  
N/A

Code Cases used: N-133, N-460, N-532-4, N-566-2, N-722-1, N-729-4, N-770-2  
(if applicable)

## CERTIFICATE OF CONFORMANCE

I certify that (a) the statements made in this report are correct; (b) the examinations and tests meet the Inspection Plan as required by the ASME Code, Section XI; and (c) the repair/replacement activities and evaluations supporting the completion of 3R18 conform to the requirements of Section XI.  
(refueling outage number)

Signed Mike H. Zulus ISI Program Owner Date 12/11/2017  
Owner or Owner's Designee, Title

## CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Connecticut and employed by The Hartford Steam Boiler Inspection and Insurance Company of Hartford, Connecticut have inspected the items described in this Owner's Activity Report, and state that, to the best of my knowledge and belief, the Owner has performed all activities represented by this report in accordance with the requirements of Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair/replacement activities and evaluation described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Inspector's Signature Elizabeth York Commissions AWI/ANII  
NB 9284 CTIBF PE, IS, C, I, N, R  
National Board, State, Province and Endorsements

Date 01-08-2018



Table 1 Items with Flaws or Relevant Conditions That Required Evaluation for Continued Service

Examination Category and Item Number	Item Description	Evaluation Description
F-A / F1.20C	Pipe Support 3-RHS-4-PSSH018	Support spring load setting evaluated by Engineering and found to be acceptable for continued service as documented in UIR MP3-17-001.
F-A / F1.10E	Pipe Support 3-RCS-1-PSSP842	Loose bolting evaluated by Engineering and found to be acceptable for continued service as documented in UIR MP3-17-002 / CR1081245. Rework performed to tighten loose bolting to restore support to design condition.
F-A / F1.10A	Pipe Support 3-RCS-1-PSR051	Loose bolting evaluated by Engineering and found to be acceptable for continued service as documented in UIR MP3-17-003 / CR1081076. Rework performed to tighten loose bolting to restore support to design condition.
F-A / F1.10C	Pipe Support 3-SIH-1-PSSH623	Support spring load setting evaluated by Engineering and found to be acceptable for continued service as documented in UIR MP3-17-004 / CR1081681.
F-A / F1.20C	Pipe Support 3-FWA-1-PSSH034	Loose bolting evaluated by Engineering and found to be acceptable for continued service as documented in UIR MP3-17-005 / CR1082413. Rework performed to tighten loose bolting to restore support to design condition.
D-B / D2.10	Valve 3CHS*V297	Evidence of leakage detected at bolted connection evaluated in accordance with ASME Code Case N-566-2 and found to be acceptable for continued service as documented in CR1047460 and ETE-MP-2016-1130 Rev. 1.
D-B / D2.10	Valve 3CHS*V340	Evidence of leakage detected at bolted connection evaluated in accordance with ASME Code Case N-566-2 and found to be acceptable for continued service as documented in CR1047433 and ETE-MP-2016-1130 Rev. 1.
D-B / D2.10	Valve 3CHS*V342	Evidence of leakage detected at bolted connection evaluated in accordance with ASME Code Case N-566-2 and found to be acceptable for continued service as documented in CR1047443 and ETE-MP-2016-1130 Rev. 1.



Table 1 Items with Flaws or Relevant Conditions That Required Evaluation for Continued Service

Examination Category and Item Number	Item Description	Evaluation Description
D-B / D2.10	Valve 3CHS*V405	Evidence of leakage detected at bolted connection evaluated in accordance with ASME Code Case N-566-2 and found to be acceptable for continued service as documented in CR1047457 and ETE-MP-2016-1130 Rev. 1.
D-B / D2.10	Valve 3CHS*V921	Evidence of leakage detected at bolted connection evaluated in accordance with ASME Code Case N-566-2 and found to be acceptable for continued service as documented in CR1047447 and ETE-MP-2016-1130 Rev. 1.
C-H / C7.10	Pump 3QSS*P3B	Evidence of leakage detected at bolted connection evaluated in accordance with ASME Code Case N-566-2 and found to be acceptable for continued service as documented in CR1052833 and ETE-MP-2016-1148.
B-P / B15.10	Valve 3RCS*V86	Evidence of leakage detected at bolted connection evaluated in accordance with ASME Code Case N-566-2 and found to be acceptable for continued service as documented in CR1083726 and ETE-MP-2017-1182.
C-H / C7.10	Pump 3RHS*P1A	Evidence of leakage detected at bolted connection evaluated in accordance with ASME Code Case N-566-2 and found to be acceptable for continued service as documented in CR1042245 and ETE-MP-2016-1104.
C-H / C7.10	Heat Exchanger 3RHS*E1A	Evidence of leakage detected at bolted connection evaluated in accordance with ASME Code Case N-566-2 and found to be acceptable for continued service as documented in CR1042244 and ETE-CME-2013-1011 Rev. 2.
C-H / C7.10	Heat Exchanger 3RSS*E1C Inlet Flange	Evidence of leakage detected at bolted connection evaluated in accordance with ASME Code Case N-566-2 and found to be acceptable for continued service as documented in CR1082792 and ETE-MP-2017-1168.



Table 1 Items with Flaws or Relevant Conditions That Required Evaluation for Continued Service

Examination Category and Item Number	Item Description	Evaluation Description
C-H / C7.10	Heat Exchanger 3RSS*E1C Outlet Flange	Evidence of leakage detected at bolted connection evaluated in accordance with ASME Code Case N-566-2 and found to be acceptable for continued service as documented in CR1082797 and ETE-MP-2017-1168.
D-B / D2.10	Valve 3SFC*V982	Evidence of leakage detected at bolted connection evaluated in accordance with ASME Code Case N-566-2 and found to be acceptable for continued service as documented in CR1077390 and ETE-MP-2017-1153.
D-B / D2.10	Pump 3SFC*P1A Discharge Flange	Evidence of leakage detected at bolted connection evaluated in accordance with ASME Code Case N-566-2 and found to be acceptable for continued service as documented in CR1077393 and ETE-MP-2017-1150.
C-H / C7.10	Valve 3SIH*V13	Evidence of leakage detected at bolted connection evaluated in accordance with ASME Code Case N-566-2 and found to be acceptable for continued service as documented in CR1048962 and ETE-MP-2016-1140.
C-H / C7.10	Flow Element 3SIH*FE968	Evidence of leakage detected at bolted connection evaluated in accordance with ASME Code Case N-566-2 and found to be acceptable for continued service as documented in CR1048964 and ETE-MP-2016-1140.
Code Case N- 729-4 / B4.10	Reactor Vessel Head Penetrations	Visual examination (VE) identified areas of historical discoloration, staining, foreign debris, minor pitting and corrosion located on the head surface and areas adjacent to numerous reactor vessel head penetrations. The subject areas have been extensively cleaned and re-inspected with no significant material degradation identified. The as-left condition of the head surface and final examination results was evaluated by Engineering to be acceptable with no active leakage present as documented in CR1081772. Subsequent examination will be performed during the next refueling outage (3R19).





## Form OAR-1 Owner's Activity Report

Attachment 1, ER-AA-ISI-100

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**Table 2 Abstract of Repair/Replacement Activities Required for Continued Service**

Code Class	Item Description	Description of Work	Date Completed	Repair/Replacement Plan Number
3	Pipe Spool	Repair Service Water Spool piece 3-SWP-24-92-3	05/17/2016	53102970112
3	Pipe Spool	Replace sections of Service Water Line 3-SWP-003-276-3	10/22/2017	53102990073
3	Pipe Spool	Repair Service water spool piece 3-SWP-20-8A	10/27/2017	53103131457
3	Pipe Spool	Repair areas of corrosion damage on inlet flange spool of 3-SWP-21-9A	10/28/2017	53102784190