

APPENDIX J

REACTOR PRESSURE VESSEL DESIGN SUMMARY REPORT - UNIT 1

J.1 INTRODUCTION

During the general update of the FSAR in 1984, the original Appendices J, K, and L were deleted and replaced with short summaries. These appendices were redocketed June 23, 1989 by the reference letter. Section J.2 is an abbreviated summary of the Unit 1 design and fabrication requirement presented in the redocketed Appendix J.

J.2 DESIGN AND FABRICATION REQUIREMENTS - SUMMARY

The Browns Ferry reactor vessel, Unit No. 1, was designed, fabricated, inspected and tested in accordance with the American Society of Mechanical Engineers Boiler and Pressure Vessel Code, Section III Nuclear Vessels, 1965 Edition and Addenda to and including Summer 1965 Addenda, and the following additions:

- a. Low alloy steel forgings for pressure parts in accordance with ASTM A-508 C1. 2 material, Code Case 1332-2.
- b. Inconel nozzles in accordance with SB-166 material, Code Case 1336, paragraph 1.
- c. Nozzle ends for austenitic pipe and flange ends for low alloy steel nozzles in accordance with SA-105 GR-II material, Code Case 1332-1.
- d. Studs, nuts, bushings, and washers in accordance with ASTM A-540, GR. B23 or 24 material, Code Case 1335-2, paragraph 4, Class 3 or 5.
- e. Shroud support legs, baffle plate, and ring in accordance with SB-168 annealed material, Code Case 1336.
- f. Core spray brackets in accordance with ASTM A-276 TP 304 material, Code Case 1334.

The date of the contract between the Buyer, General Electric Company, Atomic Power Equipment Department, San Jose, California, and the Seller, Babcock and Wilcox Company, Barberton, Ohio, was May 5, 1966. There were no deviations

from the Code throughout the design, fabrication, inspection, and testing of the reactor vessels.

REFERENCE

TVA Letter from M. J. Ray to U. S. Nuclear Regulatory Commission, Brown's Ferry Nuclear Plant, Unit 1, 2, 3, Original Final Safety Analysis Report (FSAR) Appendix J, K, and L submittals, June 23, 1989.