NIAGARA MOHAWK

NIAGARA MOHAWK POWER CORPORATION/300 ERIE BOULEVARD WEST, SYRACUSE, N.Y. 13202/TELEPHONE (315) 474-151

September 1, 1978

Director of Nuclear Reactor Regulation
Attn: Mr. Thomas Ippolito, Chief
Operating Reactors
Branch #3
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Gentlemen:

Re: Nine Mile Point Unit 1
Docket No. 50-220
DPR-63

Niagara Mohawk Power Corporation plans to install a Radwaste Reduction System and associated waste handling equipment at Nine Mile Point Unit 1.

The advantage of the Radwaste Reduction System will be to decrease the amount of solid waste that is shipped to offsite burial grounds. The System is expected to achieve an overall volume reduction factor of about 10. This will result in fewer shipments of radwaste, will extend the existing space at offsite burial grounds and result in handling fewer radwaste containers.

The Radwaste Reduction System information contained herein as Attachment A is submitted in accordance with the requirements of 10CFR20.305. Additionally, Attachment A contains description of the associated waste handling equipment for your information.

The associated waste handling equipment will be capable of solidifying waste without operation of the Radwaste Reduction System. Design of the associated handling system will proceed independently of the Nuclear Regulatory Commission review of the Radwaste Reduction System.

An evaluation was made relative to 10CFR50.59(a) and it was concluded that the Radwaste Reduction System and associated waste handling equipment do not involve an unreviewed safety question since:

A001
S11

(1) The probability of occurrence or the consequences of an accident or malfunction of equipment important to safety previously evaluated in the safety analysis report will not increase and

(2) The possibility for an accident or malfunction of a different type than any evaluated previously in the safety analysis report will not be created; and

(3) The margin of safety as defined in the basis for any technical specification will not be reduced.

These conclusions are based on the fact that none of the equipment to be contained within the radwaste building is important to safety. Analyses have been performed assuming release of the maximum amount of incinerated, but non-solidified radwaste on-hand. The results show that only the building structure is important to safety. This structure will be equal to or better than the existing radwaste building in terms of probability or consequences of failure. The building will be a seismic Class I Structure. No new accidents will be introduced other than different types of postulated handling accidents within the building structure. Releases from such accidents have been demonstrated to be insignificant when compared to failure of the building structure analyzed herein.

These modifications do not involve any changes to the Technical Specifications. No Technical Specifications are affected by these modifications.

Waste processed by the Radwaste Reduction System will be no different from those previously described in the Final Safety Analysis Report. These wastes include:

1) Filter sludges

2) Deep bed and powdered demineralizer resins

3) Concentrated waste

4) Filters, paper, wood and other combustible materials which may have been radioactively contaminated.

The Radwaste Reduction System will be housed in a building adjacent to the existing waste building on the east side of the plant. This building will also contain the solidification and handling equipment necessary to process the product for final offsite shipment. Building dimensions will be about 60 feet by 270 feet. The building will be designed to the Class I seismic requirements described in the Final Safety Analysis Report.

Any spillage of liquid waste will be controlled by the floor drains in the building. There will be no increases in liquid waste effluents to the environment due to operation of the System. The equipment will be designed to the requirements outlined in Nuclear Regulatory Commission Branch Technical Position 11-1 (Revision 1).

The final product (ash) from the Radwaste Reducation System will be processed through the solidification system into 55-gallon containers. In the event that the Radwaste Reduction System is not operating, for any reason, the waste will go directly to solidification. The solidification system will be remotely operated. It will be capable of processing any of the raw waste or the Radwaste Reduction System ash described above into a free-standing solid with no free water.

Under the present schedule, groundbreaking for the new building is to take place in February, 1979. The system is expected to be operational by October, 1980.

Very truly yours,

NIAGARA MOHAWK POWER CORPORATION

Donald P. Dise
Vice President-Engineering

LMM/szd

I. Background

The Radwaste Reduction System for radwaste volume reduction has been described in detail in the Licensing Topical Report.[1] The installation at Nine Mile Point Unit 1 will not vary significantly from the System described in this Licensing Topical Report.

As was stated in the Licensing Topical Report, the basic processes of liquid calcination and combustible waste incineration which are used in the Radwaste Reduction System has been used in industrial plants for decades. Fluidized bed calcination of radioactive wastes was developed during the period 1952-1959 at the Idaho National Engineering Laboratory. Use of calcination for liquid radwaste reduction was first demonstrated in an engineering scale facility, the Waste Calcining Facility, at the Idaho Chemical Processing Plant in 1963. The successful operation of the Waste Calcining Facility has demonstrated that liquid wastes can be routinely calcined into a granular free-flowing powder which can subsequently be handled in a simplified manner. Since 1963, the Waste Calcining Facility has handled over 2.5 million gallons of radioactive aqueous wastes which have been calcined to approximately 42,500 cubic feet of solids.

Incineration of combustible radioactive wastes has been in use as a disposal technique since 1948 when a pilot plant incinerator and offgas cleanup system were built at Mound Laboratory. Early systems were adaptations of standard refuse incinerators and did show that considerable volume reduction in waste handling was possible.

The Radwaste Reduction System is based on advanced fluidized bed technology using an inert bed medium to incinerate and calcine with a single-chamber process vessel. The purpose is to reduce the volume of the radwaste shipped offsite. Efficient volume reduction process depends upon complete combustion and effective separation of gases and solids in the effluent gas stream. This separation takes place in the offgas cleanup system. The high heat capacity of the fluidized bed gives the high temperature stability and results in very efficient combustion. The air, which maintains the bed in its fluid state, provides an ample supply of oxygen for combustion. Some wastes such as sludges and slurries do not have sufficient caloric content to maintain the bed at the desired temperature. In these cases, additional heat is provided by the combustion of supplemental fuel. The thermal inertia of the bed ensures that the system is relatively insensitive to moderate variations and caloric content of the feed. In the calcination mode, heat is used to drive off water as a vapor, leaving behind an incombustible residue. This incombustible residue is ground off the bed particles by the agitation of the bed and exits from the process vessel to a dry cyclone. The calcination process is endothermic, and heat is supplied by the combustion of supplemental fuel. The use of special inert bed material means that the bed does not have to be changed when switching from incineration to calcination.

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1. Topical Report, Radwaste Volume Reduction System, EI/NNI-77-7-P, Newport News Industrial Corporation and Energy Incorporated, June 1977.

II. System Description

The system consists of the process vessel, a dry cyclone, a product hopper, a wet scrubbing system and filtration system. Solids (ash) are removed as the gas exits from the process vessel cyclone. A product hopper collects the solids from the cyclone. Figure 1 shows the major components in a block flow diagram.

Process off-gas leaving the incinerator-calcinator vessel is cleaned in a mechanical dry cyclone, a wet scrubbing system and filtration system. The wet scrubbing system is comprised of a spray quench tank, a high energy venturi scrubber followed by a wet cyclone, a condenser, and mist eliminator. Gaseous fission products (iodines) are removed by the scrub liquid and by an adsorber in the filtration system. Particulate material is removed by the dry cyclone, wet scrub system, and high efficiency particulate absolute filters. Cleaned offgas is vented to the atmosphere (via the plant stack) while the product, a dry granular residue from the dry cyclone, is removed for solidification, storage and shipment. Scrub liquid will be processed through the liquid waste system.

The system is designed to operate at a negative pressure with respect to its surroundings, thereby providing further assurance that no leakage of radioactive material will occur. Continuous air monitors are intended to monitor the room air.

The high efficiency treatment of the offgas cleanup system minimizes the release of gaseous effluents to the atmosphere. In case a portion of the offgas cleanup system should fail to clean adequately, the Radwaste Reduction System has the capability of recirculating the offgas through the cleanup system instead of releasing it to the atmosphere. This action is initiated by the radiation monitor in the exhaust stream. There will be no liquid releases from the System directly to the environment. Scrub liquid goes to an internal hold-up tank before returning to the liquid radwaste system.

Appropriate instrumentation will be provided to detect conditions that may result in excessive radiation levels within the System. Controls designed to sense and activate an alarm upon the occurrence of a wide variety of off-normal operating conditions will be included. A part of the controls will be an annunciator panel, which will provide identification of the causes of an alarm. Corrective action will be taken either automatically or manually, depending on the potential seriousness of the occurrence. Offgas from the system is routed to the main stack. The stack monitoring system will monitor these releases. In addition, a separate system radioactivity monitor will be located in the offgas exhaust line to the plant stack. The incremental dose rates, as shown in Table 1 for normal operation, are well below the limits set in Appendix I to 10CFR50. The radioactive effluents produced by the System during normal operations will be so small that their addition to other effluents currently discharged from Nine Mile Point will have no significant environmental impact.

The solid granular residue, or product, of the System will be packaged and transported to a licensed disposal site. In accordance with Regulatory Guide 1.21, provisions will be made to monitor and to limit the radiation from each package of solid waste. This will permit the operator to control radiation exposure to personnel and to meet the regulatory requirements of 10CFR71.

III. Accident Analysis

The System as installed at Nine Mile Point Unit 1, will be in compliance with Federal Regulations concerning protection of personnel against radiation and other technical and legal licensing requirements. The system design results in very low radiation levels. The individual cubicles formed by the concrete shield walls, and the operation of the System at less than atmospheric pressure, will assure that the operational dose rate is below the levels required by 10CFR20 and are consistent with the original plant design criteria. The emissions from operation of the system result in concentrations and dose rates at the site boundary, which are well below the limiting values of 10CFR20 for unrestricted areas.

In this report two types of releases to the atmosphere are considered: normal releases from regular operations and abnormal releases due to a transient event or an accident. Because of the high efficiency of the offgas cleanup system, normal releases are inconsequential. The normal release rates from the system have been computed, and are shown in Table 1, using the maximum activities and composition shown on Table 2 and the decontamination factors from Table 4-2 of the Licensing Topical Report. The dose factors are from Regulatory Guide 1.109; a breathing rate of 20 cubic meters/day has been used. The annual dose contributions are all less than 0.001 millirem.

Exposures from transient events and accidents have been discussed in Section 4.3 of the Licensing Topical Report. No additional coverage of transients will be presented here. None of the transient events have consequences which are more severe than the maximum credible accident. As in the Licensing Topical Report, the maximum credible accident for the Nine Mile Point Unit 1 Radwaste Reduction System is the gross failure of the product container.

The doses presented for this accident are presented in Table 3. These doses are conservative since it was assumed that only 90 percent of the activity was retained by the building and ventilation system. The building housing of the System is a seismic I structure and the building ventilation discharges to the plant stack. In addition, the system will also be located in a cubicle within the radwaste building. If the product container were to catastrophically fail, much of the material would be retained inside the cubicle. The amount escaping the cubicle would be drawn into the ventilation system. The ventilation system will contain a high efficiency particulate absolute filter having a removal efficiency of 99.97 percent. Therefore, of the amount escaping the cubicle, approximately 0.03 percent would escape the filter and be discharged to the plant stack.

The capacity of the product container is equivalent to three 55-gallon drums (0.624 cubic meters), and it is conservatively calculated that 1710 curies is the maximum credible activity that can be expected to accumulate in the product container. This is based on the maximum specific activity for filter sludge shipped reported for any six months (68.5 curies/cubic meter). This occurred in the second half of 1975. This has been multiplied by a factor of 2 to allow for variations within this six-month period. Thus, it is assumed that enough feed is available at 137 curies/cubic meter to fill up the product container. The maximum volume reduction factor envisioned for waste other than dry, combustible solids is 20:1. The 1710 curies is over 2/3 of the annual expected activity for resin/sludge. It is extremely unlikely that such a large portion of the activity in a year's waste would accumulate in such a small volume. The composition of the 1710 curies is taken to be that given in the resin/sludge column of Table 2.

Despite the above, it is conservatively assumed that 10 percent of the granular ash (171 curies) in the product container escapes from the building containing the System and remains airborne long enough to reach the site boundary. The doses due to this release are shown in Table 3. The site boundary closest to Nine Mile Point Unit 1 is 1,500 meters in the southwest sector. The dilution factor, X/Q, from Regulatory Guide 1.3 for an elevated (100 meter) release and fumigation conditions are assumed. The material was assumed to be released in the first four (4) hours. These assumptions are from the latest Regulatory Guides and are therefore different from the assumptions used in the Nine Mile Point Unit 1 Final Safety Analysis Report. The dose factors have been taken from Regulatory Guide 1.109, and the breathing rate was 20 cubic meters/day. The maximum dose was found to be 534 mrem to the lung.

IV. Associated Waste Handling Equipment

A solidification system will be added to solidify wastes processed by the System and the existing radwaste facility. It will consist of dry cement storage and handling equipment, 55-gallon barrel filling, mixing equipment, and settling/decant tanks. This equipment will be of the same basic design as that at Dresden 2 and 3.

An overhead crane will be installed for transporting barrels from the mixing station to storage and for loading them onto trucks for shipment and offsite burial.

Existing wastes such as filter sludges, resins and evaporator bottoms will either be processed by the system as described earlier or will be solidified directly. These wastes, if solidified directly, will be settled and decanted to the desired concentration. They will then be mixed with a premeasured amount of cement in a 55-gallon barrel. A test will be conducted after solidification to ensure that no free water is present.

The 55-gallon barrels will then be transported by the crane to the storage area(s). A barrel grab mechanism equipped with TV cameras will be used to pick up and locate the barrels in the storage areas. However, the storage areas will be inaccessible to personnel and the entire operation will be remotely operated. Adequate shielding will ensure that radiation levels in normal plant access areas are consistent with the existing plant design. In addition, the roof over the barrel storage areas will be two (2) feet thick to ensure acceptable radiation levels outside the building.

The crane described above will have access from the barrel storage area(s) to the truck loading bay. Barrels handled by the crane will be located by position in the storage area, picked up and placed in a cask on a truck. These operations will be remotely controlled.

Controls for the system as well as alarms and monitoring equipment important for the operation of the system will be located in a control room in the new building. Each barrel in storage will have a number assigned corresponding to its location. Other information on each barrel will also be recorded on a board in the control room such as radiation level, weight and date placed in storage.

TABLE 1

ADDITIONAL EMISSION RATES, BOUNDARY CONCENTRATIONS, AND DOSE RATES DUE TO OPERATION OF THE RADWASTE REDUCTION SYSTEM

	Maximum Feed Rate (Ci/year)	Decontamination Factor	Release Rate (Ci/year)	Concentration Limit (pCi/m^3)	Maximum Boundary Concentration (pCi/m^3)	Dose Rates Thyroid	Lung (mrem/yr)	Total
NA-24	15	4×10^4	3.8×10^{-4}	5000	6.7×10^{-7}	8.3×10^{-9}	8.3×10^{-9}	8.3 x
Mn-54	125	4×10^4	3.1×10^{-3}	1000	5.5×10^{-6}	0.0	7.0×10^{-6}	3.2 x
Co-60	915	4×10^4	2.3×10^{-2}	300	4.0×10^{-5}	0.0	2.2×10^{-4}	5.5 x
Sr-89	10	4×10^4	2.5×10^{-4}	300	4.4×10^{-7}	0.0	5.6×10^{-7}	3.5 x
I-131	50	1×10^4	5.0×10^{-3}	100	8.8×10^{-6}	9.6×10^{-5}	0.0	1.7 x
Cs-134	1225	4×10^4	3.06×10^{-2}	400	5.4×10^{-5}	0.0	4.8×10^{-6}	3.6 x
Cs-137	2160	4×10^4	5.4×10^{-2}	500	9.5×10^{-5}	0.0	6.6×10^{-6}	3.7 x
TOTAL	4500		1.16×10^{-1}		2.1×10^{-4}	9.5×10^{-5}	2.4×10^{-4}	7.4 x

TABLE 2

PROJECTED ACTIVITIES IN THE LIQUID AND RESIN/SLUDGE FEED
TO THE RADWASTE REDUCTION SYSTEM FOR NINE MILE POINT UNIT ONE

	Liquid			Resin/Sludge		
	(percent)	Expected (Ci/yr)	Maximum (Ci/yr)	(percent)	Expected (Ci/yr)	Maximum (Ci/yr)
Na-24	1.5	9	15			
Mn-54	2	12	20	3	75	105
Co-60	11	66	110	23	575	805
Sr-89	1	6	10			
I-131	1.5	9	15	1	25	35
Cs-134	35	210	350	25	625	875
Cs-137	48	288	480	48	1200	1680
Total		600	1000		2500	3500

TABLE 3

DOSES AT THE SITE BOUNDARY DUE TO THE MAXIMUM CREDIBLE ACCIDENT
FOR THE NINE MILE POINT UNIT #1 RADWASTE REDUCTION SYSTEM

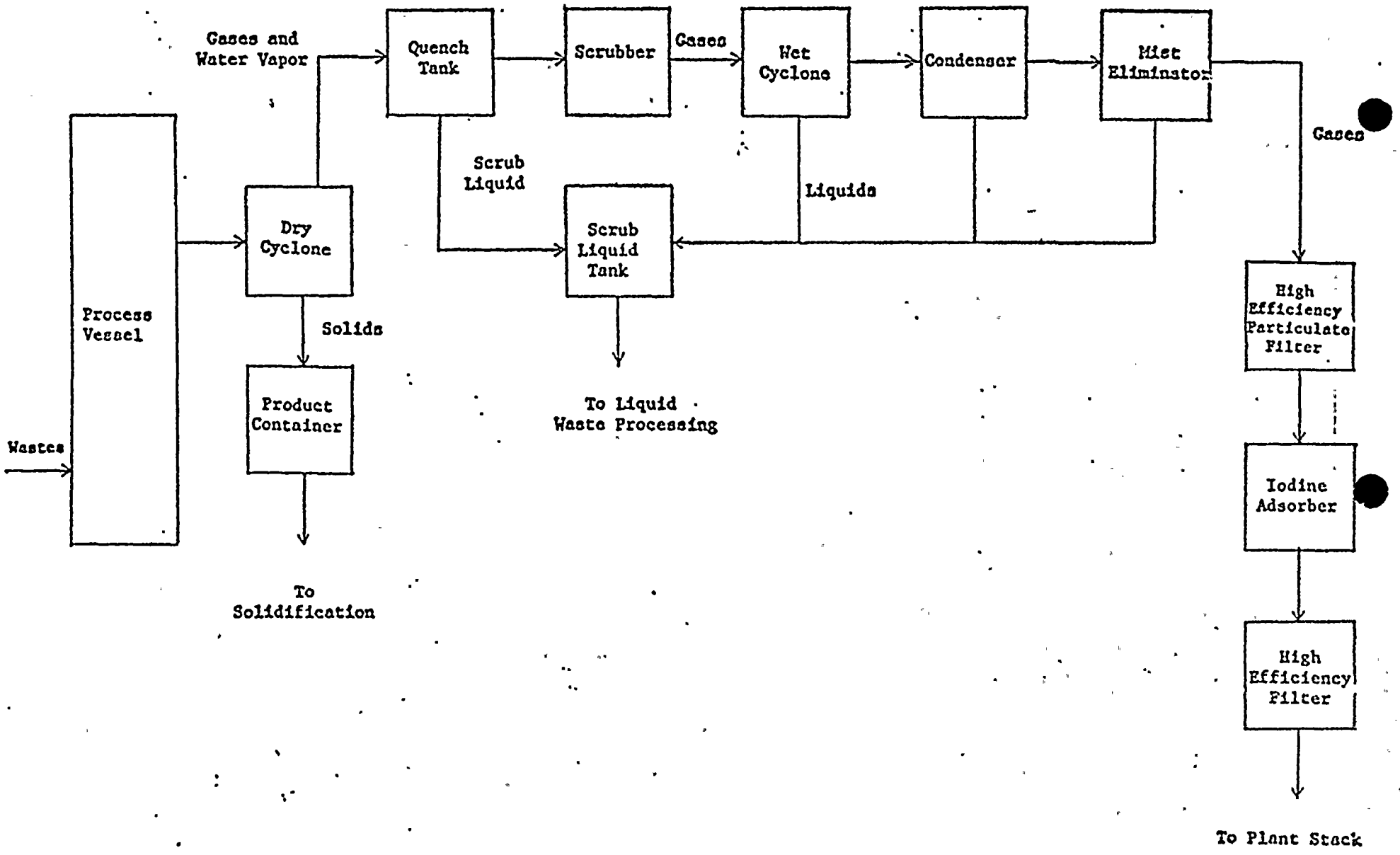
10% of the Ash Released

Organ Dose (mrem)

Nuclide	Bone	Liver	Thyroid	Kidney	Lung	Total Body
Mn- 54	0.0	0.4	0.0	0.1	15.3	0.1
Co- 60	0.0	1.0	0.0	0.0	496.5	1.2
I-131	0.1	0.1	43.2	0.2	0.0	0.1
Cs-134	33.5	76.5	0.0	26.1	8.7	65.7
Cs-137	82.8	107.7	0.0	38.7	13.2	74.4
TOTAL	116.4	185.7	43.2	65.1	533.7	141.5

Figure #1

Radwaste Reduction System Block Flow Diagram



FROM:

Thomas B. Cochran
Natural Resources Defense Council, Inc.

TO:

Secy of the Commission

ACTION CONTROL	DATES	CONTROL NO. 04774
COMPL. DEADLINE	[illegible]	
ACKNOWLEDGMENT		DATE OF DOCUMENT
INTERIM REPLY		
FINAL REPLY	for Director	PREPARE FOR SIGNATURE OF: ☐ CHAIRMAN ☐ EXECUTIVE DIRECTOR
FILE LOCATION	11/15/78	OTHER: Director

DESCRIPTION ☐ LETTER ☐ MEMO ☐ REPORT ☐ OTHER

Req info re the health & safety consequences of operating a planned low level waste incinerator at the Three Mile Island Unit 1

SPECIAL INSTRUCTIONS OR REMARKS

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JCAE NOTIFICATION RECOMMENDED: ☐ YES ☐ NO

NRC FORM 232
(11-75)

EXECUTIVE DIRECTOR FOR OPERATIONS

PRINCIPAL CORRESPONDENCE CONTROL

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NRC SECRETARIAT

TO:
- ☐ Commissioner _____ Date _____
- ☒ Exec. Dir./Oper.
- ☐ Cong. Liaison
- ☐ Public Affairs
- ☐ Gen. Counsel
- ☐ Solicitor
- ☐ Secretary
- ☐ _____

Incoming: Thomas B. Cochran
From: NRDC

To: The Secretary Date 10/25/78
Subject: Requests technical information on Nine Mile Point unit No. 1

- ☐ Prepare reply for signature of:
  - ☐ Chairman
  - ☐ Commissioner _____
  - ☐ EDO, GC, CL, SOL, PA, SECY
  - ☒ Signature block omitted
  - ☐ _____
- ☒ Return original of incoming with response Rec'd 0000 EDO Date 10/27/78 Time 2:10

- ☒ For direct reply*
- ☐ For appropriate action
- ☐ For information
- ☐ For recommendation

SECY SUSPENSE: Nov. 10, 1978

Remarks: Cy to SECY, ~~OGC~~, OGC.
Original & Docket

For the Commission: White

*Send three (3) copies of reply to Secy Mail Facility

NRC-62 ACTION SLIP

Natural Resources Defense Council, Inc.

917 15TH STREET, N.W.
WASHINGTON, D.C. 20005
202 737-5000

Western Office
2345 YALE STREET
PALO ALTO, CALIF. 94306
415 327-1080

New York Office
122 EAST 42ND STREET
NEW YORK, N.Y. 10017
212 949-0049

October 25, 1978

U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Attn: Secretary of the Commission

Dear Sir,

Please provide me with any technical information you have describing the health ahd safety consequences of operating the low level waste incinerator planned for installation at Nine Mile Point Unit No. 1 in New York.

Have you, or do you intend to prepare an environmental assessment of this technology? If not, why not? If so, please send me a copy when it becomes available.

Given the public concern over the installation of this incinerator, an environmental impact statement, or at least an environmental assessment would appear appropriate.

Sincerely,

Thomas B. Cochran

TBC/ps

Distribution
Docket
ORB #3
SSheppard

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555

September 22, 1978

Docket No.. 50-220

See attached list of addressees

Subject: Nine Mile Point Unit No. 1
Niagara Mohawk Power Corporation

The following documents concerning our review of the subject facility are transmitted for your information:

- ☐ Notice of Receipt of Application.
- ☐ Draft/Final Environmental Statement, dated _____.
- ☐ Safety Evaluation, or Supplement No. _____, dated _____.
- ☐ Notice of Hearing on Application for Construction Permit.
- ☐ Notice of Consideration of Issuance of Facility Operating License.
- ☐ Application and Safety Analysis Report, Vol. _____.
- ☐ Amendment No. _____ to Application/SAR, dated _____.
- ☐ Construction Permit No. CPPR-_____, dated _____.
- ☐ Facility Operating License No. DPR-_____, NPF-_____, dated _____.
- ☐ Amendment No. _____ to CPPR-_____ or DRR-_____, dated _____.
- ☒ Other: Ltr dtd 8/31/78 from NMPC transmitting Radioactive Effluent Release Semi-Annual Report for the period 1/1/78 through 6/30/78 w/att meteorological data.
- ☐

Office of Nuclear Reactor Regulation
ORB #3
DOR

Enclosures:
As stated

ENVIRO2

cc:

OFFICE ▶	ORB #3					
SURNAME ▶	SSheppard:mjf					
DATE ▶	9/21/78					

NRC-21 (6-76)

Chief
Division of Ecological Services
Bureau of Sport Fisheries and Wildlife
U. S. Department of the Interior
Washington, D. C. 20240

Director
National Oceanographic Data Center
Environmental Data Service
National Oceanic & Atmospheric Administration
U. S. Department of Commerce
Washington, D. C. 20235

Dr. James T. Tanner
National Bureau of Standards
Reactor Building 235
Washington, D. C. 20234

Headquarters

Dr. Neill Thomasson (AW-459)
Chief, Energy Systems Analysis Branch
Office of Radiation Programs
Environmental Protection Agency
401 M Street S.W.
Washington, D.C. 20460

EPA Region

U. S. Environmental Protection Agency
Region II Office
ATTN: EIS COORDINATOR
26 Federal Plaza
New York, N. Y. 10007

Distribution
Docket
ORB #3
SSheppard

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555

July 28, 1978

Docket No. 50-220

See attached list of addressees

Subject: Nine Mile Point Unit No. 1
Niagara Mohawk Power Company

The following documents concerning our review of the subject facility are transmitted for your information:

- ☐ Notice of Receipt of Application.
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- ☐ Facility Operating License No. DPR-_____, NPF-_____, dated_____.
- ☐ Amendment No. _____ to CPPR-_____ or DRR-_____, dated _____.
- ☒ Other: Ltr dtd 7/21/78 from Niagara Mohawk Power Co transmitting 1977 Nine Mile Point Aquatic Ecology Studies
- ☐

Office of Nuclear Reactor Regulation
ORB #3
DOR

Enclosures:
As stated

cc:

ENVIRO 2

OFFICE ▶	ORB #3					
SURNAME ▶	SSheppard:mjf					
DATE ▶	7/26/78					

NRC-21 (6-76)

Chief
Division of Ecological Services
Bureau of Sport Fisheries and Wildlife
U. S. Department of the Interior
Washington, D. C. 20240

Director
National Oceanographic Data Center
Environmental Data Service
National Oceanic & Atmospheric Administration
U. S. Department of Commerce
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Dr. James T. Tanner
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Headquarters

Dr. Neill Thomasson (AW-459)
Chief, Energy Systems Analysis Branch
Office of Radiation Programs
Environmental Protection Agency
401 M Street S.W.
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EPA Region

U. S. Environmental Protection Agency
Region II Office
ATTN: EIS COORDINATOR
26 Federal Plaza
New York, N. Y. 10007

Lab

Dr. Philip F. Gustafson
Argonne National Laboratory
9700 South Cass Avenue
Argonne, Illinois 60439

Distribution
Docket
ORB #3
SSheppard

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555

May 11, 1978

Docket No. 50-220

See attached list of addressees

Subject: Nine Mile Point Unit No. 1
Niagara Mohawk Power Corporation

The following documents concerning our review of the subject facility are transmitted for your information:

- ☐ Notice of Receipt of Application.
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- ☐ Safety Evaluation, or Supplement No. _____, dated _____.
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- ☐ Facility Operating License No. DPR-_____, NPF-_____, dated _____.
- ☐ Amendment No. _____ to CPPR-_____ or DRR-_____, dated _____.
- ☒ Other: Ltr dtd 5/3/78 from Niagara Mohawk Pwr Corp. transmitting an addendum to the Annual Environ. Operating Rpt. for the period 1/1/77 through 12/31/77.
- ☐

Office of Nuclear Reactor Regulation
ORB #3
DOR

Enclosures:
As stated

cc:

OFFICE ▶	ORB #3					
SURNAME ▶	SSheppard:mjf					
DATE ▶	5/11/78					

NRC-21 (6-76)

Chief
Division of Ecological Services
Bureau of Sport Fisheries and Wildlife
U. S. Department of the Interior
Washington, D. C. 20240

Director
National Oceanographic Data Center
Environmental Data Service
National Oceanic & Atmospheric Administration
U. S. Department of Commerce
Washington, D. C. 20235

Dr. James T. Tanner
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Washington, D.C. .20460

EPA Region

U. S. Environmental Protection Agency
Region II Office
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New York, N. Y. 10007

LAB

Dr. Philip F. Gustafson
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Argonne, Illinois 60439

CENTRAL FILES

UNITED STATES
NUCLEAR REGULATORY COMMISSION
REGION I
631 PARK AVENUE
KING OF PRUSSIA, PENNSYLVANIA 19406

MAR 31 1977

Mr. Paul A. Giardina, Chief
Regional Office of Radiation Programs
U.S. Environmental Protection Agency
Region II
26 Federal Plaza
New York, New York 10007

Dear Mr. Giardina:

This is in response to your letter to Dr. C. O. Gallina of this office, dated March 23, 1977, relative to effluent and solid waste reporting by the James A. FitzPatrick and Nine Mile Point Unit 1 plants.

The licensee, Niagara Mohawk Power Corporation, submits the effluent and waste disposal reports semi-annually as separate documents and makes the same distribution as for the Annual Report and Environmental Operating Report, which you indicated had been received by your office.

The Effluent and Waste Disposal reports for the period of interest have been received by this office and are dated as follows:

Plant	Period	Report Date
JAF	Jan-Jun 1976	August 31, 1976
JAF	July-Dec 1976	February 23, 1977
NMP-1	Jan-Jun 1976	August 30, 1976
NMP-1	July-Dec 1976	February 23, 1977

If this information does not prove helpful in locating these reports from your usual source, please contact this office.

Eldon J. Brunner, Chief
Reactor Operations and Nuclear
Support Branch

UNITED STATES ENVIRONMENTAL PROTECTION AGENCY
REGION II
26 FEDERAL PLAZA
NEW YORK, NEW YORK 10007

March 23, 1977

Mr. Charles Gallina
Region I
Office of Inspection and Enforcement
U.S. Nuclear Regulatory Commission
631 Park Avenue
King of Prussia, Pennsylvania 19406

Dear Mr. Gallina:

The Regional Office of Radiation Programs has received the Annual Environmental Operating Reports for the Nine Mile Point Nuclear Station Unit #1 and for the James A. Fitzpatrick Nuclear Power Plant covering the period January 1, 1976 through December 31, 1976, and the Annual Report of Operation for the Nine Mile Point Station for the same period.

According to Paragraph (a)(2) of Section 50.36a, 10 CFR Part 50, the Nuclear Regulatory Commission requires reports from utilities which specify the quantity of each of the principal radionuclides released in liquid and gaseous effluents during the operation of a nuclear power plant. This effluent release and solid waste data, as well as dose readings were not contained in these reports we received. We would appreciate any information you can provide concerning these apparent omissions.

Sincerely yours,

Paul A. Giardina
Chief
Regional Office of Radiation Programs

Distribution
Docket
ORB #3
CParrish

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555

March 2, 1977

Docket No. 50-220

SEE ATTACHED LIST OF ADDRESSEES

Subject: Nine Mile Point Nuclear Station Unit No. 1 located in Oswego County, New York

The following documents concerning our review of the subject facility are transmitted for your information:

☐ Notice of Receipt of Application.

☐ Draft/Final Environmental Statement, dated _____.

☐ Safety Evaluation, or Supplement No. _____, dated _____.

☐ Notice of Hearing on Application for Construction Permit.

☐ Notice of Consideration of Issuance of Facility Operating License.

☐ Application and Safety Analysis Report, Vol. _____.

☐ Amendment No. _____ to Application/SAR, dated _____.

☐ Construction Permit No. CPPR-_____, dated _____.

☐ Facility Operating License No. DPR-_____, NPF-_____, dated _____.

☐ Amendment No. _____ to CPPR-_____ or DRR-_____, dated _____.

☒ Other: Ltrs. from Niagara Mohawk Power Corp (1) dtd 2/24/77 transmitting Annual Report of Operations for the period 1/1/76 thru 12/31/76 and (2) dtd 2/25/77 transmitting Annual Environmental Operating Report for the period 1/1/76 thru 12/31/76.

☐

Office of Nuclear Reactor Regulation
Operating Reactors Branch #3
Division of Operating Reactors

Enclosures:
As stated

cc:

EC

OFFICE ▶	ORB #3 CP					
SURNAME ▶	CParrish:mjf					
DATE ▶	3/2/77					

NRC-21 (6-76)

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Chief
Division of Ecological Services
Bureau of Sport Fisheries and Wildlife
U. S. Department of the Interior
Washington, D. C. 20240

Director
National Oceanographic Data Center
Environmental Data Service
National Oceanic & Atmospheric Administration
U. S. Department of Commerce
Washington, D. C. 20235

Director
Division of Chemical Technology
Food and Drug Administration
U. S. Department of Health
Education and Welfare
Washington, D. C. 20240

Headquarters

Dr. Neill Thomasson (AW-459)
Chief, Energy Systems Analysis Branch
Office of Radiation Programs
Environmental Protection Agency
401 M Street S.W.
Washington, D.C. 20460

EPA Region

U. S. Environmental Protection Agency
Region II Office
ATTN: EIS COORDINATOR
26 Federal Plaza
New York, N. Y. 10007

LAB

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Argonne, Illinois 60439