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DESCRIPTION

ENCLOSURE

PLANT NAME:

NOTE: IF PERSONNEL EXPOSURE IS INVOLVED
SEND DIRECTLY TO KREGER/J. COLLINS

FOR ACTION/INFORMATION

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W/3 CYS FOR ACTION

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INTERNAL DISTRIBUTION

☒ REG FILE

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EISENHUT

BAER

SHAO

VOLLMER/BUNCH

KREGER/J. COLLINS

EXTERNAL DISTRIBUTION

CONTROL NUMBER

LPDR: Morris, L

TIC:

NSIC:

770760149

100

100

100

100

() ()



Commonwealth Edison
Dresden Nuclear Power Station
R.R. #1
Morris, Illinois 60450
Telephone 815/942-2920

BBS Ltr. 200-77

March 11, 1977

REGULATORY DOCKET FILE COPY

Mr. James G. Keppler, Regional Director
Directorate of Regulatory Operations - Region III
U. S. Nuclear Regulatory Commission
799 Roosevelt Road
Glen Ellyn, Illinois 60137



Enclosed please find Reportable Occurrence report number 50-237/1976-74.
This report is being submitted to your office in accordance with the Dresden
Nuclear Power Station Technical Specifications, Section 6.6.B.

B. B. Stephenson
Station Superintendent
Dresden Nuclear Power Station

BBS:jo

Enclosure

cc: Director of Inspection & Enforcement
Director of Management Information & Program Control
File/NRC

770760149

PLEASE PRINT ALL REQUIRED INFORMATION

**EVENT
TYPE**

GPO 881-667

EVENT DESCRIPTION (Continued)

Analysis indicated that the reactivity insertion was substantially below the limit of $0.013\Delta K$ required by Tech Spec section 3.3.B.3.a., and that the equivalent stable period was 10 ± 20 seconds. Furthermore, the withdrawal was performed in accordance with G.E. control rod withdrawal sequencing rules for reactor power levels below 20%. Since the reactor protection systems functioned as designed, this event was considered to be of little safety significance. This is the first reported occurrence of this nature at Dresden. (50-237/1976-74)

CAUSE DESCRIPTION (Continued)

the reactor before the operator could reinsert the control rod or adjust instrument ranges.

Following a normal scram recovery, start-up operations were resumed. Although the notch withdrawal of CRD J-2 was postponed until a later step in the control rod withdrawal sequence, all CRD movements remained in keeping with the G.E. start-up CRD withdrawal sequencing rules described above. Additionally, the nuclear engineers discussed the incident and developed methods of avoiding future potentially undesirable notch pulls under similar conditions.

As the result of an administrative oversight, this event was initially classified as a non-reportable occurrence. On 2/25/77, the station was notified that the occurrence was reportable because it represented a short-term reactivity increase corresponding to a reactor period of less than 5 seconds (Tech Spec section 6.6.B.1.d). As a result of this incident, administrative and supervisory personnel were reacquainted with this particular reporting requirement.

RECEIVED DOCUMENT
PROCESSING UNIT

1977 MAR 16 PM 3 51



Commonwealth Edison
Dresden Nuclear Power Station
R.R. #1
Morris, Illinois 60450
Telephone 815/942-2920

D. LANHAM

BBS Ltr. 200-77

March 11, 1977

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Station Superintendent
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Director of Management Information & Program Control
File/NRC

MAR 15 1977



2 3 4 5

2 3 4 5

LICENSEE EVENT REPORT

CONTROL BLOCK:

[PLEASE PRINT ALL REQUIRED INFORMATION]

LICENSEE NAME	LICENSE NUMBER	LICENSE TYPE	EVENT TYPE
01 I L D R S 2	00-000000-00	41111	01
7 8 9 14	15 25	26 30	31 32

CONT	CATEGORY	REPORT TYPE	REPORT SOURCE	DOCKET NUMBER	EVENT DATE	REPORT DATE
01	 	T	L	050-0237	122876	031077
7 8	57 58	59	60	61 68	69 74	75 80

EVENT DESCRIPTION

02 During start-up operations, control rod J-2 was withdrawn one notch. The reactivity

03 inserted by this notch withdrawal resulted in a transient power increase. A

04 reactor scram on IRM high flux prevented the transient from attaining a measur-

05 able stable period; however, period instrumentation indicated transient readings

06 of approximately 5 seconds. (Continued)

SYSTEM CODE	CAUSE CODE	COMPONENT CODE	PRIME COMPONENT SUPPLIER	COMPONENT MANUFACTURER	VIOLATION
07 Z Z	F	Z Z Z Z Z Z	Z	Z 9 9 9	Y
7 8 9 10	11	12 17	43	44 47	48

CAUSE DESCRIPTION

08 A combination of existing conditions such as the control rod pattern, moderator

09 density, and local xenon concentration apparently resulted in an unexpectedly

10 high reactivity worth for the notch. The resulting transient increase tripped

(Continued)

FACILITY STATUS	% POWER	OTHER STATUS	METHOD OF DISCOVERY	DISCOVERY DESCRIPTION
11 C	000	NA	A	Reactor Short Period, Scram
7 8 9	10 12	13	44 45	46 80

FORM OF ACTIVITY RELEASED	CONTENT OF RELEASE	AMOUNT OF ACTIVITY	LOCATION OF RELEASE
12 Z	Z	NA	NA
7 8 9	10 11	44	45 80

PERSONNEL EXPOSURES

NUMBER	TYPE	DESCRIPTION
13 0 0 0	Z	NA
7 8 9 11	12	13 80

PERSONNEL INJURIES

NUMBER	DESCRIPTION
14 0 0 0	NA
7 8 9 11	12 80

OFFSITE CONSEQUENCES

15 NA

LOSS OR DAMAGE TO FACILITY

TYPE	DESCRIPTION
16 Z	NA
7 8 9	A10 80

PUBLICITY

17 NA

ADDITIONAL FACTORS

18 NA

19

NAME: James G. Toscas

PHONE: Ext. 464

EVENT DESCRIPTION (Continued)

Analysis indicated that the reactivity insertion was substantially below the limit of $0.013\Delta K$ required by Tech Spec section 3.3.B.3.a., and that the equivalent stable period was 10 - 20 seconds. Furthermore, the withdrawal was performed in accordance with G.E. control rod withdrawal sequencing rules for reactor power levels below 20%. Since the reactor protection systems functioned as designed, this event was considered to be of little safety significance. This is the first reported occurrence of this nature at Dresden. (50-237/1976-74)

CAUSE DESCRIPTION ((Continued))

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(1) UNION (2) STATE

UNITED STATES