

50-237

NRC DISTRIBUTION FOR PART 50 DOCKET MATERIAL

FILE NUMBER

INCIDENT REPORT

TO: Mr. James G. Keppler

FROM: Commonwealth Edison Company
Morris, Illinois
B B Stephenson

DATE OF DOCUMENT

3/24/77

DATE RECEIVED

3/31/77

☒ LETTER
☐ ORIGINAL
☒ COPY

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PROP

INPUT FORM

NUMBER OF COPIES RECEIVED

100

DESCRIPTION

Ltr. trans the following:

DO NOT REMOVE

(1-P)

PLANT NAME:

Dresden Unit No. 2.

RJL

ACKNOWLEDGED

ENCLOSURE

Licensee Event Report (RO 50-237/1977-7) on
2/23/77 concerning the tech spec requirement
for the IRM trip setpoint being exceeded....

(2-P)

NOTE: IF PERSONNEL EXPOSURE IS INVOLVED
SEND DIRECTLY TO KREGER/J. COLLINS

FOR ACTION/INFORMATION

BRANCH CHIEF:

W/3 CYS FOR ACTION

LIC. ASST.:

W/7 CYS

ACRS 16 CYS HOLDING/SENT

INTERNAL DISTRIBUTION

☒ REG FILE

NRC PDR

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TEDESCO/MACCARY

EISENHUT

BAER

SHAO

VOLLMER/BUNCH

KREGER/J. COLLINS

EXTERNAL DISTRIBUTION

LPDR: Morris, Ill

TIC:

NSIC:

CONTROL NUMBER

770910306

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1999



Commonwealth Edison
Dresden Nuclear Power Station
R.R. #1
Morris, Illinois 60450
Telephone 815/942-2920

REGULATORY DOCKET FILE COPY

BBS Ltr. #77-250

March 24, 1977



Mr. James G. Keppler, Regional Director
Directorate of Regulatory Operations - Region III
U. S. Nuclear Regulatory Commission
799 Roosevelt Road
Glen Ellyn, Illinois 60137

Enclosed please find Reportable Occurrence report number 50-237/1977-7.
This report is being submitted to your office in accordance with the Dresden
Nuclear Power Station Technical Specifications, Section 6.6.B.

B. B. Stephenson
Station Superintendent
Dresden Nuclear Power Station

BBS:jo

Enclosure

cc: Director of Inspection & Enforcement
Director of Management Information & Program Control
File/NRC

1951-1952

CONTROL BLOCK:

PLEASE PRINT ALL REQUIRED INFORMATION

LICENSEE NAME														LICENSE NUMBER												LICENSE TYPE						EVENT TYPE	
01	1	1	D	R	S	2	0	0	-	0	0	0	0	0	-	0	0	4	1	1	1	1	0	3									
7	8	9	14	15	25	26	30	31	32																								
01 CONT		CATEGORY		REPORT TYPE	REPORT SOURCE	DOCKET NUMBER						EVENT-DATE						REPORT DATE															
01	CONT			L	L	0	5	0	-	0	2	3	7	0	2	2	3	7	7	0	3	2	5	7	7								
7	8	57	58	59	60	61	68	69	74	75																							

EVENT DESCRIPTION

02	During routine weekly surveillance testing, IRM #16 was found to trip at 122% of																								80
03	scale; IRM #16's rod block setpoint was found at 111%. The Tech Specs require																								80
04	the IRM trip setpoint to be less than or equal to 120%, and the rod block setpoint to																								80
05	be less than or equal to 108%. Since all other IRM channels were operable,																								80
06	each reactor trip system retained the required number of instrument channels.																								80

SYSTEM CODE		CAUSE CODE		COMPONENT CODE						PSME COMPONENT SUPPLIER		COMPONENT MANUFACTURER				VIOLATION		(Continued)
07	I	A	E	I	N	S	T	R	U	N	G	0	8	0	Y			
7	8	9	10	11	12	17	43	44	47	48								

CAUSE DESCRIPTION

08	The event was attributed to instrument setpoint drift. The trip and rod block set-																								80
09	points for IRM #16 were adjusted to the correct settings. IRM #16 will continue																								80
10	to be tested weekly to ensure that any significant setpoint variations are identified																								80

FACILITY STATUS		% POWER		OTHER STATUS						METHOD OF DISCOVERY		DISCOVERY DESCRIPTION						(Continued)
11	E	0	9	3	NA						B	NA						
7	8	9	10	12	13	44	45	46										

FORM OF ACTIVITY RELEASED		CONTENT OF RELEASE		AMOUNT OF ACTIVITY						LOCATION OF RELEASE					
12	Z	Z	NA						NA						
7	8	9	10	11	44	45									

PERSONNEL EXPOSURES

NUMBER		TYPE		DESCRIPTION								
13	0	0	0	Z	NA							
7	8	9	11	12	13							

PERSONNEL INJURIES

NUMBER		DESCRIPTION									
14	0	0	0	NA							
7	8	9	11	12							

OFFSITE CONSEQUENCES

15	NA																								80
----	----	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	----

LOSS OR DAMAGE TO FACILITY

TYPE		DESCRIPTION								
16	Z	NA								
7	8	9	10							

PUBLICITY

17	NA																								80
----	----	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	----

ADDITIONAL FACTORS

18	NA																								80
----	----	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	----

19																									80
----	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	----

NAME: Desi Santanna

PHONE: Ext. 421

CPO 601-607

EVENT DESCRIPTION (Continued)

This is not a repetitive occurrence. (50-237/1977-7)

CAUSE DESCRIPTION (Continued)

in a timely manner. No further action is considered appropriate.

RECEIVED DOCUMENT
PROCESSING UNIT

1977 MAR 31 PM 12 17



Commonwealth Edison
Dresden Nuclear Power Station
R.R. #1
Morris, Illinois 60450
Telephone 815/942-2920

D. Latham

BBS Ltr. #77-255

March 24, 1977

Mr. James G. Keppler, Regional Director
Directorate of Regulatory Operations - Region III
U. S. Nuclear Regulatory Commission
799 Roosevelt Road
Glen Ellyn, Illinois 60137

Enclosed please find Reportable Occurrence report number 50-237/1977-8.
This report is being submitted to your office in accordance with the Dresden
Nuclear Power Station Technical Specifications, Section 6.6.B.

B. B. Stephenson
Station Superintendent
Dresden Nuclear Power Station

BBS:jo

Enclosure

cc: Director of Inspection & Enforcement
Director of Management Information & Program Control
File/NRC

MAR 29 1977

SECRET

SECRET

SECRET

LICENSEE EVENT REPORT

CONTROL BLOCK: 1 6

[PLEASE PRINT ALL REQUIRED INFORMATION]

LICENSEE NAME														LICENSE NUMBER												LICENSE TYPE						EVENT TYPE			
01	I	L	D	R	S	2	0	0	-	0	0	0	0	0	-	0	0	4	1	1	1	1	0	3											
7	8	9				14	15											25	26					30	31	32									
CATEGORY		REPORT TYPE		REPORT SOURCE		DOCKET NUMBER												EVENT DATE						REPORT DATE											
01	CONT			L	L	0	5	0	-	0	2	3	7	0	2	2	5	7	7	0	3	2	5	7	7										
7	8		57	58	59	60	61					68	69								74	75				80									

EVENT DESCRIPTION

02	During routine surveillance testing, main steam line low pressure isolation switch	80
03	2-261-30C was found to trip at 853 psi (Tech Specs require a trip point greater	80
04	than or equal to 855 psi, including a 5-psi head correction). Redundant instrumenta-	80
05	tion would have provided the required isolation signal had a low pressure condi-	80
06	tion occurred. A similar event involving PS 2-261-30C occurred in March, 1975.	80

(50-23744977-8)

SYSTEM CODE		CAUSE CODE		COMPONENT CODE				PRIME COMPONENT SUPPLIER		COMPONENT MANUFACTURER				VIOLATION	
07	C	D	E	I	N	S	T	R	U	N	B	0	8	0	Y
7	8	9	10	11	12				17	43	44			47	48

CAUSE DESCRIPTION

08	The event was attributed to instrument setpoint drift. PS 2-261-30C was reset to	80
09	trip at the correct setpoint. The switch will continue to be tested monthly to	80
10	ensure that reportable setpoint variations are identified and corrected in a timely	80

(Continued)

FACILITY STATUS		% POWER		OTHER STATUS				METHOD OF DISCOVERY		DISCOVERY DESCRIPTION					
11	E	0	9	5	NA				B	NA					
7	8	9	10	11	12	13			44	45	46			80	
FORM OF ACTIVITY RELEASED		CONTENT OF RELEASE		AMOUNT OF ACTIVITY				METHOD OF DISCOVERY		LOCATION OF RELEASE					
12	Z	Z	NA				NA		NA						
7	8	9	10	11					44	45				80	

PERSONNEL EXPOSURES

NUMBER		TYPE		DESCRIPTION	
13	0	0	Z	NA	
7	8	9	11	12	13

PERSONNEL INJURIES

NUMBER		DESCRIPTION		
14	0	0	0	NA
7	8	9	11	12

OFFSITE CONSEQUENCES

15	NA	80
----	----	----

LOSS OR DAMAGE TO FACILITY

TYPE		DESCRIPTION	
16	Z	NA	
7	8	9	10

PUBLICITY

17	NA	80
----	----	----

ADDITIONAL FACTORS

18	CAUSE DESCRIPTION (Continued) manner. PS 2-261-30C is a Barksdale	80
19	pre-cycled pressure switch, type B2T-A1255.	80

NAME: T. E. Lang

PHONE: Ext. 222



1-1

THE JOURNAL OF THE
ROYAL ANTHROPOLOGICAL INSTITUTE



Commonwealth Edison
Dresden Nuclear Power Station
R.R. #1
Morris, Illinois 60450
Telephone 815/942-2920

D. LANHAM

BBS Ltr. #77-250

March 24, 1977

Mr. James G. Keppler, Regional Director
Directorate of Regulatory Operations - Region III
U. S. Nuclear Regulatory Commission
799 Roosevelt Road
Glen Ellyn, Illinois 60137

Enclosed please find Reportable Occurrence report number 50-237/1977-7.
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B. B. Stephenson
Station Superintendent
Dresden Nuclear Power Station

BBS:jo

Enclosure

cc: Director of Inspection & Enforcement
Director of Management Information & Program Control
File/NRC

MAR 29 1977

LICENSEE EVENT REPORT

CONTROL BLOCK: 1 6

PLEASE PRINT ALL REQUIRED INFORMATION

LICENSEE NAME														LICENSE NUMBER												LICENSE TYPE						EVENT TYPE	
01	1	1	D	R	S	2	0	0	-	0	0	0	0	-	0	0	4	1	1	1	1	0	3										
7	8	9				14	15									25	26						31	32									

CATEGORY		REPORT TYPE		REPORT SOURCE		DOCKET NUMBER						EVENT DATE						REPORT DATE								
01	CONT			L	L	0	5	0	-	0	2	3	7	0	2	2	3	7	7	0	3	2	5	7	7	
7	8			57	58	59	60						68	69						74	75					80

EVENT DESCRIPTION

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SYSTEM CODE		CAUSE CODE		COMPONENT CODE				PRIME COMPONENT SUPPLIER		COMPONENT MANUFACTURER				VIOLATION		(Continued)
07	I	A	E	I	N	S	T	R	U	N	G	0	8	0	Y	
7	8	9	10	11					17	43				47	48	

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FACILITY STATUS		% POWER		OTHER STATUS		METHOD OF DISCOVERY		DISCOVERY DESCRIPTION		(Continued)
11	E	0	9	3	NA	B	NA			
7	8	9	10	12	13	44	45	46		80

FORM OF ACTIVITY RELEASED		CONTENT OF RELEASE		AMOUNT OF ACTIVITY		LOCATION OF RELEASE	
12	Z	Z	NA		NA		
7	8	9	10	11	44	45	80

PERSONNEL EXPOSURES

NUMBER		TYPE		DESCRIPTION	
13	0	0	Z	NA	
7	8	9	11	12	13

PERSONNEL INJURIES

NUMBER		DESCRIPTION	
14	0	0	NA
7	8	9	12

OFFSITE CONSEQUENCES

15	NA	80
----	----	----

LOSS OR DAMAGE TO FACILITY

TYPE		DESCRIPTION	
16	Z	NA	
7	8	9	10

PUBLICITY

17	NA	80
----	----	----

ADDITIONAL FACTORS

18	NA	80
----	----	----

19		80
----	--	----

NAME: Desi Santanna PHONE: Ext. 421

11. 1990.

EVENT DESCRIPTION (Continued)

This is not a repetitive occurrence. (50-237/1977-7)

CAUSE DESCRIPTION (Continued)

in a timely manner. No further action is considered appropriate.



10-10-10

10-10-10