

50-237

## NRC DISTRIBUTION FOR PART 50 DOCKET MATERIAL

FILE NUMBER  
INCIDENT REPORT

TO: Mr. James G. Keppler

FROM: Commonwealth Edison Company  
Morris, Illinois  
B. B. Stephenson

DATE OF DOCUMENT  
4/15/77DATE RECEIVED  
4/21/77

☒ LETTER  
☐ ORIGINAL  
☒ COPY

☐ NOTORIZED  
☒ UNCLASSIFIED

PROP

INPUT FORM

NUMBER OF COPIES RECEIVED

1 cc.

## DESCRIPTION

Ltr. trans the following:

PLANT NAME:  
Dresden Unit No. 2

RJL

(1-P)

## ENCLOSURE

Licensee Event Report (RO. 50-237/1977-13) on  
4/1/77 concerning routine surveillance of the  
drywell floor drain sump pump flow rate on  
Unit No. 2 indicating a leak of reactor  
coolant inside the primary containment....

ACKNOWLEDGED

DO NOT REMOVE

NOTE: IF PERSONNEL EXPOSURE IS INVOLVED  
SEND DIRECTLY TO KREGER/J. COLLINS

## FOR ACTION/INFORMATION

BRANCH CHIEF:

ZEIMANN

W/3 CYS FOR ACTION

LIC. ASST.:

DIGGS

W/3 CYS

ACRS 16 CYS HOLDING/SENT AS CAT. B

## INTERNAL DISTRIBUTION

☒ REG FILE  
☐ NRC PDR  
☐ I & E (2)  
☐ MIPC  
☐ SCHROEDER/IPPOLITO  
☐ HOUSTON  
☐ NOVAK/CHECK  
☐ GRIMES  
☐ CASE  
☐ BUTLER  
☐ HANAUER  
☐ TEDESCO/MACCARY  
☐ EISENHUT  
☐ BAER  
☐ SHAO  
☐ VOLLMER/BUNCH  
☐ KREGER/J. COLLINS

## EXTERNAL DISTRIBUTION

LPDR: MORRIS ILL.  
TIC:  
NSIC:

## CONTROL NUMBER

7711570138





**Commonwealth Edison**  
Dresden Nuclear Power Station  
R.R. #1  
Morris, Illinois 60450  
Telephone 815/942-2920

BBS Ltr. #77-322

April 15, 1977

Mr. James G. Keppler, Regional Director  
Directorate of Regulatory Operations - Region III  
U. S. Nuclear Regulatory Commission  
799 Roosevelt Road  
Glen Ellyn, Illinois 60137



Enclosed please find Reportable Occurrence report number 50-237/1977-13.  
This report is being submitted to your office in accordance with the Dresden  
Nuclear Power Station Technical Specifications, Section 6.6.B.

  
B. B. Stephenson  
Station Superintendent  
Dresden Nuclear Power Station

BBS:jo

Enclosure

cc: Director of Inspection & Enforcement  
Director of Management Information & Program Control  
File/NRC

771150138





Commonwealth Edison

## DEVIATION REPORT

DVR NO.  
STA UNIT YEAR NO.  
D-12 - 2 - 77 - 29

## 1 TITLE OF DEVIATION

OCCURRED

4/1/77

DATE

TIME

Leak in 4" Bypass Loop B Water Chemistry Test Flange

SYSTEM AFFECTED

PLANT CONDITIONS

TESTING

Reactor

MODE S/U

PWR (MWT) 0

LOAD (MWE) 0

YES

NO

DESCRIPTION OF EVENT

Excessive drywell leakage of approximately 2200 gal./8 hrs. caused unit shutdown.

Tech Spec limit is 2400 gal./8 hrs.

DESCRIPTION OF CAUSE

4" bypass loop B water chemistry test flange conductivity cell electrical seal blew out.

OTHER APPLICABLE INFORMATION

This was determined during drywell leak inspection.

EQUIPMENT ☒ YES  
FAILURE ☐ NODR NO.  
NAWR NO.  
3074

J. S. Kolanowski

RESPONSIBLE SUPERVISOR

4/1/77

DATE

## PART 2 OPERATING ENGINEERS COMMENTS

The leak was located during drywell leak inspection. Leakage was minimal and

within the capability of the floor drain pump however, Tech Spec limit was being

approached. The test flange will be removed and original blank flange will be reinstalled.

TYPE OF DEVIATION  
REPORTABLE OCCURRENCEEVENT OF POTENTIAL  
PUBLIC INTERESTTECH SPEC  
VIOLATIONNON-REPORTABLE  
OCCURRENCEANNUAL  
REPORTINGSAFETY-  
RELATED  
WR ISSUED

14 DAY

☒

30 DAY

☐☐☐☐YES ☒NO ☐REPORTABLE OCCURRENCE  
NUMBER

ACTION ITEM NO.

PROMPT ON-SITE NOTIFICATION

50-237 - 77 - 13

NA

A. Roberts - Asst. Supt.

4/1/77

1500

TITLE

DATE

TIME

NA

TITLE

DATE

TIME

24-HOUR NRC NOTIFICATION

PROMPT OFF-SITE NOTIFICATION

☒ TPH

W. Shafer

4/1/77

16:10

REGION III

DATE

TIME

☐ TGM

NA

REGION III &amp; DOL

DATE

TIME

F. Palmer

4/4/77

10:02

TITLE

DATE

TIME

J. Gilliom

4/4/77

10:02

TITLE

DATE

TIME

REVIEW AND COMPLETED

S. Kolanowski

4/1/77

DATE

ACCEPTANCE BY STATION REVIEW  
AS REQUIRED

DATE

RESOLUTION APPROVED AND

C.E. Sargent

4/15/77

S. Kolanowski

4/15/77

4/15/77



# LICENSEE EVENT REPORT

CONTROL BLOCK: 1 6

PLEASE PRINT ALL REQUIRED INFORMATION

LICENSEE NAME										LICENSE NUMBER										LICENSE TYPE					EVENT TYPE		
01	I	L	D	R	S	2	0	0	-	0	0	0	0	0	-	0	0	4	1	1	1	1	0	1			
7	8	9				14	15											25	26				30	31	32		
CATEGORY		REPORT TYPE		REPORT SOURCE		DOCKET NUMBER										EVENT DATE					REPORT DATE						
01	CONT				T	L	0	5	0	-	0	2	3	7	0	4	0	1	7	7	0	4	1	5	7	7	
7	8		57	58	59	60	61							68	69							74	75				80

**EVENT DESCRIPTION**

02	Routine surveillance of the drywell floor drain sump pump flow rate on Dresden																								80
03	Unit-2 indicated a leak of reactor coolant inside the primary containment. Even																								80
04	though the allowable drywell leakage as specified by the Technical Specifications																								80
05	had not been exceeded, the unit was shutdown. A visual inspection inside the																								80
06	drywell on April 1, 1977, revealed that a pressure seal contained in a 1/2-inch																								80

(Continued)

SYSTEM CODE				CAUSE CODE		COMPONENT CODE						PRIME COMPONENT SUPPLIER		COMPONENT MANUFACTURER				VIOLATION	
07	C	B	E	X	X	X	X	X	X	N	G	0	8	0	Y				
7	8	9	10	11	12					17	43	44			47	48			

**CAUSE DESCRIPTION**

08	The test flange was removed and replaced by the original blank flange. The																								80
09	associated blank flange for the "A" reactor recirculation pump discharge valve																								80
10	bypass piping was inspected satisfactorily. The General Electric Company has																								80

(Continued)

FACILITY STATUS			% POWER			OTHER STATUS			METHOD OF DISCOVERY			DISCOVERY DESCRIPTION									
11	D	0	0	0	NA	C	NA														
7	8	9	10	11	12	13	44	45	46							80					
FORM OF ACTIVITY RELEASED			CONTENT OF RELEASE			AMOUNT OF ACTIVITY			LOCATION OF RELEASE												
12	Z	Z	NA	NA	NA	NA															
7	8	9	10	11	44	45										80					

**PERSONNEL EXPOSURES**

NUMBER			TYPE		DESCRIPTION											
13	0	0	0	Z	NA											
7	8	9	11	12	13											80

**PERSONNEL INJURIES**

NUMBER			DESCRIPTION													
14	0	0	0	NA												
7	8	9	11	12												80

**OFFSITE CONSEQUENCES**

15	NA															
7	8	9														80

**LOSS OR DAMAGE TO FACILITY**

TYPE		DESCRIPTION														
16	Z	NA														
7	8	9	10													80

**PUBLICITY**

17	NA															
7	8	9														80

**ADDITIONAL FACTORS**

18	NA															
7	8	9														80

19																
7	8	9														80

NAME: John W. Wuiciga

PHONE: Ext. 421





EVENT DESCRIPTION (Continued)

instrumentation pipe had failed. The pipe penetrated a water chemistry test flange located in the 4-inch bypass piping around reactor recirculation pump discharge valve M0 2-0202-58. The test flange had been installed to permit chemically analyzing the reactor water in the bypass piping during operation. This testing program was completed at the end of the last fuel cycle. This event is not repetitive. (50-237/1977-13)

CAUSE DESCRIPTION (Continued)

requested that the test flange which failed, be sent to them for further investigation. In view of the fact that test flanges of the type that failed are not presently installed, no further corrective action is considered necessary.

