NRC FORM 195 (2-76)

U.S. NUCLEAR REGULATORY COMMISSION

DOCKET NUMBER 50-237

FILE NUMBER INCIDENT REPORT

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TO: Mr. Norman C. Moseley	FROM: Commonwealth Edison Morris, Illinois 60450 B. B. Stephenson	DATE OF DOCUMENT 05-09-77 DATE RECEIVED 05-16-77

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DESCRIPTION

Ltr. Trans The Following:

(1 page)

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PLANT NAME: DRESDEN UNIT # 2

jcm

ACKNOWLEDGED

ENCLOSURE

Licensee Event Report (RO-50-237/77-19) on 04-25-77 concerning APRM/RBM flow-bias indication exceeded 100% on both channel's A and B.....

(2 pages)

NOTE: IF PERSONNEL EXPOSURE IS INVOLVED SEND DIRECTLY TO KREGER/J. COLLINS

FOR ACTION/INFORMATION

BRANCH CHIEF:	Davis
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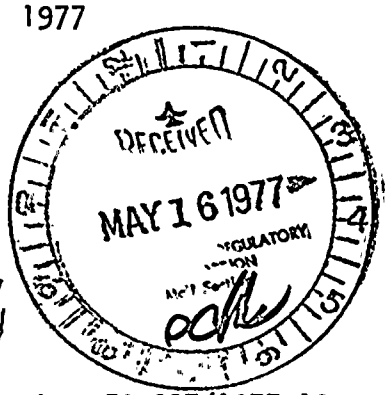
CONTROL NUMBER

771370023

Commonwealth Edison
Dresden Nuclear Power Station
R.R. #1
Morris, Illinois 60450
Telephone 815/942-2920

BBS Ltr. # 77-419

May 9, 1977



Mr. James G. Keppler, Regional Director
Directorate of Regulatory Operations - Region III
U. S. Nuclear Regulatory Commission
799 Roosevelt Road
Glen Ellyn, IL 60137

Enclosed please find Reportable Occurrence report number 50-237/1977-19. This report is being submitted to your office in accordance with the Dresden Nuclear Power Station Technical Specifications, Section 6.6.B.

B. B. Stephenson
Station Superintendent
Dresden Nuclear Power Station

BBS:sm

Enclosure

cc: Director of Inspection & Enforcement
Director of Management Information & Program Control
File/NRC

LICENSEE EVENT REPORT

CONTROL BLOCK: [ ] (1–6) (PLEASE PRINT ALL REQUIRED INFORMATION)

	LICENSEE NAME	LICENSE NUMBER	LICENSE TYPE	EVENT TYPE
01	ILDRS2	00-00000-00	4111	01
(7–8)	(9–14)	(15–25)	(26–30)	(31–32)

	CATEGORY	REPORT TYPE	REPORT SOURCE	DOCKET NUMBER	EVENT DATE	REPORT DATE
01 CONT		T	L	050-0237	042577	050977
(7–8)	(57–58)	(59)	(60)	(61–68)	(69–74)	(75–80)

EVENT DESCRIPTION

02 During normal operation it was discovered that with core flow at 100%, the APRM/RBM
03 flow-bias indication exceeded 100% on both channels A and B. At 100% core flow, the
04 APRM scram setpoint (117% reactor power) would not have been exceeded; however, at
05 reduced flow rates, the flow-biased scram setpoint would have been non-conservative
06 by approximately 3%. The flow bias flow indication has exceeded 100% in the past (continued)

	SYSTEM CODE	CAUSE CODE	COMPONENT CODE	PRIME COMPONENT SUPPLIER	COMPONENT MANUFACTURER	VIOLATION
07	IA	D	INSTRU	N	G080	Y
(7–8)	(9–10)	(11)	(12–17)	(43)	(44–47)	(48)

CAUSE DESCRIPTION

08 During the recent outage, the recirculation drive flow instruments and the APRM/RBM
09 flow converters which receive their input from the instruments were calibrated to
10 appropriate original specifications. The procedure which was followed, however, did (continued)

	FACILITY STATUS	% POWER	OTHER STATUS	METHOD OF DISCOVERY	DISCOVERY DESCRIPTION
11	E	083	NA	B	NA
(7–8)	(9)	(10–12)	(13–44)	(45)	(46–80)

	FORM OF ACTIVITY RELEASED	CONTENT OF RELEASE	AMOUNT OF ACTIVITY	LOCATION OF RELEASE
12	Z	Z	NA	NA
(7–8)	(9)	(10)	(11–44)	(45–80)

PERSONNEL EXPOSURES

	NUMBER	TYPE	DESCRIPTION
13	000	Z	NA
(7–8)	(9–11)	(12)	(13–80)

PERSONNEL INJURIES

	NUMBER	DESCRIPTION
14	000	NA
(7–8)	(9–11)	(12–80)

OFFSITE CONSEQUENCES

15 NA
(7–8) (9–80)

LOSS OR DAMAGE TO FACILITY

	TYPE	DESCRIPTION
16	Z	NA
(7–8)	(9)	(10–80)

PUBLICITY

17 NA
(7–8) (9–80)

ADDITIONAL FACTORS

18 NA
(7–8) (9–80)

19 [ ]
(7–8) (9–80)

NAME: T. Rausch PHONE: Ext. 266

GPO 881-444

EVENT DESCRIPTION (continued)

(50-237/1977-2) as a result of calibration of the total core flow indication. The safety implications of this event were minimal because safety and transient analyses do not assume a flow-biased scram function (50-237/1977-19).

CAUSE DESCRIPTION (continued)

not take into account the present correlation between recirculation drive flow and total core flow (to which the APRM/RBM flow converter readings are referenced). As a result the APRM/RBM scram trip levels could have been non-conservative at core flow rates of less than 100%.

On April 25, 1977, the APRM/RBM flow converters were readjusted to read 100% at 100% total core flow. To prevent a reoccurrence of this event, the appropriate instrument calibration procedure will be revised to include the present correlation between recirculation drive flow and total core flow.