

50-237

NRC DISTRIBUTION FOR PART 50 DOCKET MATERIAL

FILE NUMBER
INCIDENT REPORT

TO: Mr. James G. Keppler

FROM: Commonwealth Edison Company
Morris, Illinois
B. B. Stephenson

DATE OF DOCUMENT
5/9/77DATE RECEIVED
5/23/77

☒ LETTER
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☒ UNCLASSIFIED

PROP

INPUT FORM

NUMBER OF COPIES RECEIVED

1516NED

DESCRIPTION

DO NOT REMOVE

PLANT NAME:

Dresden Unit No. 2

RJL

ACKNOWLEDGED
(1-P)

ENCLOSURE

Licensee Event Report (RO 50-237/1977-19)
on 4/25/77 concerning the APRM/RBM flow-
bias indication exceeding 100% on both
channels A & B.....

(2-P)

NOTE: IF PERSONNEL EXPOSURE IS INVOLVED
SEND DIRECTLY TO KREGER/J. COLLINS

FOR ACTION/INFORMATION

BRANCH CHIEF: ZEIMANN

W/3 CYS FOR ACTION

LIC. ASST.: DIGGS

W/1 CYS

ACRS 16 CYS HOLDING/SENT AS CAT B

INTERNAL DISTRIBUTION

☒ REG FILE

☐ NRC-PDR

☐ I & E (2)

☐ MIPG

☐ SCHROEDER/IPPOLITO

☐ HOUSTON

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☐ GRIMES

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☐ EISENHUT

☐ BAER

☐ SHAO

☐ VOLLMER/BUNGH

☐ KREGER/J. COLLINS

EXTERNAL DISTRIBUTION

LPDR: MORRIS 144.

TIC:

NSIC:

CONTROL NUMBER

771430052

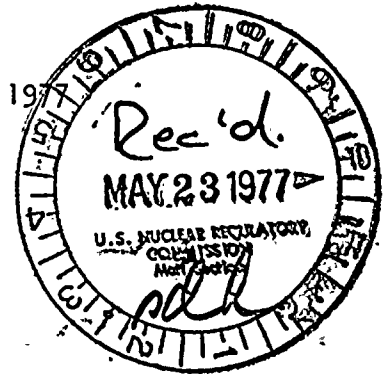


Commonwealth Edison
Dresden Nuclear Power Station
R.R. #1
Morris, Illinois 60450
Telephone 815/942-2920

D. Latham

BBS Ltr. # 77-419

May 9, 1977



Mr. James G. Keppler, Regional Director
Directorate of Regulatory Operations - Region III
U. S. Nuclear Regulatory Commission
799 Roosevelt Road
Glen Ellyn, IL 60137

REGULATORY DOCKET FILE COPY

Enclosed please find Reportable Occurrence report number 50-237/1977-19.
This report is being submitted to your office in accordance with the Dresden
Nuclear Power Station Technical Specifications, Section 6.6.B.

B. B. Stephenson
B. B. Stephenson
Station Superintendent
Dresden Nuclear Power Station

BBS:sm

Enclosure

cc: Director of Inspection & Enforcement
Director of Management Information & Program Control
File/NRC

8 51

MAY 11 1977

771430052



1918

LICENSEE EVENT REPORT

CONTROL BLOCK:

[PLEASE PRINT ALL REQUIRED INFORMATION]

LICENSEE NAME		LICENSE NUMBER										LICENSE TYPE					EVENT TYPE						
01	I	L	D	R	S	2	0	0	-	0	0	0	0	-	0	0	4	1	1	1	1	0	1
7	8	9				14	15						25	26				30	31	32			

CATEGORY		REPORT TYPE		REPORT SOURCE		DOCKET NUMBER										EVENT DATE					REPORT DATE				
01	CONT			T	L	0	5	0	-	0	2	3	7	0	4	2	5	7	7	0	5	0	9	7	7
7	8	57	58	59	60	61						68	69				74	75				80			

EVENT DESCRIPTION

02	During normal operation it was discovered that with core flow at 100%, the APRM/RBM																							80
03	flow-bias indication exceeded 100% on both channels A and B. At 100% core flow, the																							80
04	APRM scram setpoint (117% reactor power) would not have been exceeded; however, at																							80
05	reduced flow rates, the flow-biased scram setpoint would have been non-conservative																							80
06	by approximately 3%. The flow bias flow indication has exceeded 100% in the past																							80

(continued)

SYSTEM CODE		CAUSE CODE		COMPONENT CODE								PRIME COMPONENT SUPPLIER		COMPONENT MANUFACTURER					VIOLATION	
07	I	A	D	I	N	S	T	R	U	N	G	0	8	0	Y					
7	8	9	10	11	17					43	47			48						

CAUSE DESCRIPTION

08	During the recent outage, the recirculation drive flow instruments and the APRM/RBM																							80
09	flow converters which receive their input from the instruments were calibrated to																							80
10	appropriate original specifications. The procedure which was followed, however, did																							80

(continued)

FACILITY STATUS		% POWER		OTHER STATUS								METHOD OF DISCOVERY		DISCOVERY DESCRIPTION										
11	E	0	8	3	NA								B	NA										
7	8	9	10	12	13								44	45	46			80						

FORM OF ACTIVITY RELEASED		CONTENT OF RELEASE		AMOUNT OF ACTIVITY								LOCATION OF RELEASE									
12	Z	Z	NA								NA										
7	8	9	10	11	44								45	80							

PERSONNEL EXPOSURES

NUMBER		TYPE		DESCRIPTION											
13	0	0	0	Z	NA										
7	8	9	11	12	13										80

PERSONNEL INJURIES

NUMBER		DESCRIPTION												
14	0	0	0	NA										
7	8	9	11	12										80

OFFSITE CONSEQUENCES

15	NA																							80
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LOSS OR DAMAGE TO FACILITY

TYPE		DESCRIPTION																						
16	Z	NA																						
7	8	9	10																					80

PUBLICITY

17	NA																							80
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ADDITIONAL FACTORS

18	NA																							80
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19																								80
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NAME: T. Rausch

PHONE: Ext. 266

EVENT DESCRIPTION (continued)

(50-237/1977-2) as a result of calibration of the total core flow indication. The safety implications of this event were minimal because safety and transient analyses do not assume a flow-biased scram function (50-237/1977-19).

CAUSE DESCRIPTION (continued)

not take into account the present correlation between recirculation drive flow and total core flow (to which the APRM/RBM flow converter readings are referenced). As a result the APRM/RBM scram trip levels could have been non-conservative at core flow rates of less than 100%.

On April 25, 1977, the APRM/RBM flow converters were readjusted to read 100% at 100% total core flow. To prevent a reoccurrence of this event, the appropriate instrument calibration procedure will be revised to include the present correlation between recirculation drive flow and total core flow.

END PAGE 50 IN 8 51

RECEIVED DOCUMENT
PROCESSING UNIT

1977 MAY 24 AM 8 21