

## NRC DISTRIBUTION FOR PART 50 DOCKET MATERIAL

TO: J.G. KEPPLER

FROM: COMMONWEALTH EDISON CO  
MORRIS, ILLINOIS+

FILE NUMBER

INCIDENT REPORT

DATE OF DOCUMENT

7/1/77

DATE RECEIVED

7/13/77

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PROP

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1 SIGNED

## DESCRIPTION

LICENSEE EVENT REPORT FOR R.O.# 77-21, ON 6/3/77  
CONCERNING A CONTAINMENT COOLING SVC WATER  
LPCI HEAT EXCHANGER VALVE 2-1501-3A FAILING TO  
OPEN WHEN "A" CCSW PUMP WAS STARTED.

(1P &amp; 2P)

PLANT NAME: DRESDEN # 2  
SAB

## ENCLOSURE

ACKNOWLEDGED  
DO NOT REMOVENOTE: IF PERSONNEL EXPOSURE IS INVOLVED  
SEND DIRECTLY TO KREGER/J. COLLINS

## FOR ACTION/INFORMATION

BRANCH CHIEF:

W/ 3 CYS FOR ACTION

LIC ASST.:

DAVIS (4)

## INTERNAL DISTRIBUTION

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TEDESCO

EISENHIT

BAER

SHAO

VOLLMER/RUNCH

KREGER/ J. COLLINS

ROSA

## EXTERNAL DISTRIBUTION

LPDR: Morris, J. C.

TIC:

NSIC:

ACRS (16) SENT AS CAT. B

## CONTROL NUMBER

771940445

Handwritten marks in the top right corner.

SECRET  
NO 001 00 100 00

100

100

Handwritten marks in the middle left area.

SECRET  
NO 001 00 100 00  
Handwritten marks in the middle right area.



Commonwealth Edison  
Dresden Nuclear Power Station  
R.R. #1  
Morris, Illinois 60450  
Telephone 815/942-2920

*D. L. Latham*

July 1, 1977

Regulatory

File Cy.

BBS Ltr. #597-77

Mr. James G. Keppler, Regional Director  
Directorate of Regulatory Operations - Region III  
U. S. Nuclear Regulatory Commission  
799 Roosevelt Road  
Glen Ellyn, Illinois 60137



Dear Mr. Keppler:

Enclosed please find Reportable Occurrence Report  
Number 50-237/1977-21. This report is being submitted to your  
office in accordance with the Dresden Nuclear Power Station  
Technical Specifications, Section 6.6.B.

Sincerely,

*B. B. Stephenson*

B. B. Stephenson  
Station Superintendent  
Dresden Nuclear Power Station

BBS:SS:gt

Enclosure

cc: Director of Inspection & Enforcement  
Director of Management Information & Program Control  
File/NRC

771940445

JUL 6 1977



# LICENSEE EVENT REPORT

CONTROL BLOCK: 1 2 3 4 5 6

[PLEASE PRINT ALL REQUIRED INFORMATION]

LICENSEE NAME						LICENSE NUMBER						LICENSE TYPE			EVENT TYPE									
01	I	L	D	R	S	2	0	0	-	0	0	0	0	-	0	0	4	1	1	1	1	0	3	
7	8	9				14	15									25	26					30	31	32

  

CATEGORY		REPORT TYPE	REPORT SOURCE	DOCKET NUMBER				EVENT DATE				REPORT DATE											
01	CONT	L	L	0	5	0	-	0	2	3	7	0	6	0	3	7	7	0	7	0	1	7	7
7	8	57	58	59	60	61					68	69			74	75							80

**EVENT DESCRIPTION**

02	During normal plant operation, a containment cooling service water (CCSW)																							80
03	LPCI heat exchanger valve 2-1501-3A failed to open when "A" CCSW pump was started.																							80
04	The safety implications of this failure were minimal because operating surveillance																							80
05	DOS 1500-3 was performed in order to assure that a redundant CCSW LPCI heat exchanger																							80
06	loop was operable at the time of the incident. A similar occurrence involving this																							80

(Continued)

SYSTEM CODE		CAUSE CODE	COMPONENT CODE				PRIME COMPONENT SUPPLIER	COMPONENT MANUFACTURER			VOLATION					
07	S	F	A	C	O	N	T	R	L	N	B	1	3	5	N	
7	8	9	10	11	12	13	14	15	16	17	43	44	45	46	47	48

**CAUSE DESCRIPTION**

08	Just prior to finding the failed valve, tube leaks had been discovered in																							80
09	the 2A LPCI heat exchanger. In order to perform the required maintenance, valve																							80
10	2-1501-3A, the discharge valve on the 2A LPCI heat exchanger, had to be in the closed																							80

(Continued)

FACILITY STATUS		% POWER	OTHER STATUS		METHOD OF DISCOVERY		DISCOVERY DESCRIPTION			
11	E	0	7	5	NA	B	NA	NA		
7	8	9	10	11	12	13	44	45	46	80

  

FORM OF ACTIVITY RELEASED		CONTENT OF RELEASE	AMOUNT OF ACTIVITY		LOCATION OF RELEASE		
12	Z	Z	NA	NA	NA	NA	
7	8	9	10	11	44	45	80

**PERSONNEL EXPOSURES**

NUMBER	TYPE	DESCRIPTION				
13	0	0	0	Z	NA	
7	8	9	11	12	13	80

**PERSONNEL INJURIES**

NUMBER	DESCRIPTION				
14	0	0	0	NA	
7	8	9	11	12	80

**OFFSITE CONSEQUENCES**

15	NA	NA	
7	8	9	80

**LOSS OR DAMAGE TO FACILITY**

TYPE	DESCRIPTION			
16	Z	NA		
7	8	9	10	80

**PUBLICITY**

17	NA		
7	8	9	80

**ADDITIONAL FACTORS**

18	NA		
7	8	9	80

19	NA		
7	8	9	80

NAME: Desi Santanna

PHONE: X-265



EVENT DISCRIPTION (Continued)

valve last occurred in 1975. (50-237/1977-21)

CAUSE DESCRIPTION (Continued)

position. While manually closing the valve, the stem was bent. While performing maintenance on the valve stem, the feedback potentiometer (which is mechanically linked to the valve stem) inside the limitorque valve operator became misadjusted. Thus, the valve could not respond properly to demand signals from the remote valve position controller. Adjustment of the feedback potentiometer allowed the valve to function properly. The feedback potentiometer is a Model 5611 R1K(1)L.5 Helipot manufactured by Beckman Instruments, Inc.. The valves are 12"-300 PSI USA standard valve assemblies manufactured by Copes-Vulcan, a division of Blaw-Knox Co.

An investigation is being conducted in order to determine long-term corrective action. The results of this investigation will be submitted in a supplemental report by August 1, 1977.

