

NRC DISTRIBUTION FOR PART 50 DOCKET MATERIAL

FILE NUMBER

INCIDENT REPORT

DATE OF DOCUMENT
8/1/77DATE RECEIVED
8/24/77TO: Mr. James G. Keppler
FROM: Commonwealth Edison
Morris, Illinois
B. B. Stephenson☒ LETTER
☒ ORIGINAL
☐ COPY☐ NOTORIZED
☒ UNCLASSIFIED

PROP

INPUT FORM

NUMBER OF COPIES RECEIVED

1 SIGNED CY RECEIVED

DESCRIPTION

PLANT NAME: Dresden Unit No. 2 (1-P)
RJL 8/25/77

ENCLOSURE

Licensee Event Report (RO 50-237/1977-21) on
6/3/77 concerning a containment cooling
service water LPCI heat exchanger valve
2-1501-3A failing to open when "A" CCSW pump
was started.....**ACKNOWLEDGED**
DO NOT REMOVENOTE: IF PERSONNEL EXPOSURE IS INVOLVED
SEND DIRECTLY TO KREGER/J. COLLINS

FOR ACTION/INFORMATION

BRANCH CHIEF: (4)
W/ 3 CYS FOR ACTION
LIC ASST.

DAVIS

INTERNAL DISTRIBUTION

REG FILE
NRC PDR
I & E (2)
MIPC
SCHROEDER/IPPOLITO
HOUSTON
NOVAK/CHECK
GRIMES
KNIGHT
BUTLER
HANAUER
TEDESCO
EISENHUT
BAER
SHAO
VOLLMER/BUNCH
KREGER/ J. COLLINS
ROSA

EXTERNAL DISTRIBUTION

LPDR: MORRIS, III
TIC:
NSIC:
ACRS (16) SENT AS CAT. B

CONTROL NUMBER

772370121

1977

1978

1979

1980



1981

1982



Commonwealth Edison
Dresden Nuclear Power Station
R.R. #1
Morris, Illinois 60450
Telephone 815/942-2920

D. Landon

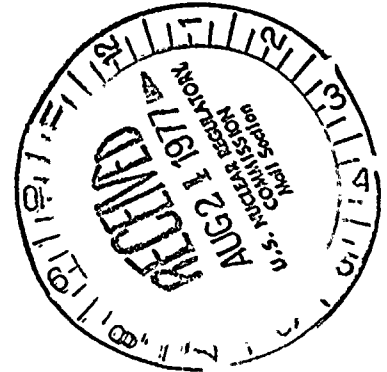
August 1, 1977

Regulatory

File-Cy

BBS LTR #77-695

Mr. James G. Keppler, Regional Director
Directorate of Regulatory Operations - Region III
U.S. Nuclear Regulatory Commission
799 Roosevelt Road
Glen Ellyn, IL 60137



Enclosed please find an update report on Reportable Occurrence report number 50-237/1977-21. This report is being submitted to your office in accordance with the Dresden Nuclear Power Station Technical Specifications, Section 6.6.B.

Arthur M. Roberts
for B.B. Stephenson
Station Superintendent
Dresden Nuclear Power Station

BBS:dlz

Enclosure

cc: Director of Inspection & Enforcement
Director of Management Information & Program Control
File/NRC

AUG 3 1977

772370121

LICENSEE EVENT REPORT

CONTROL BLOCK: 1 6

(PLEASE PRINT ALL REQUIRED INFORMATION)

LICENSEE NAME										LICENSE NUMBER										LICENSE TYPE					EVENT TYPE	
01 I L D R S 2										0 0 - 0 0 0 0 0 0 - 0 0										4 1 1 1 1					0 3	
7	8	9								14	15						25	26				30	31	32		

CATEGORY		REPORT TYPE	REPORT SOURCE	DOCKET NUMBER					EVENT DATE					REPORT DATE					
01 CONT		L	L	0 5 0 - 0 2 3 7					0 6 0 3 7 7					0 7 0 1 7 7					
7	8	57	58	59	60	61					68	69				74	75	80	

EVENT DESCRIPTION

02	7	8	9	During normal plant operation, a containment cooling service water (CCSW)																																																																														80
03	7	8	9	LPCI heat exchanger valve 2-1501-3A failed to open when "A" CCSW pump was started.																																																																														80
04	7	8	9	The safety implications of this failure were minimal because operating surveillance																																																																														80
05	7	8	9	DOS 1500-3 was performed in order to assure that a redundant CCSW LPCI heat exchanger																																																																														80
06	7	8	9	loop was operable at the time of the incident. A similar occurrence involving this																																																																														80

(Continued)

SYSTEM CODE		CAUSE CODE	COMPONENT CODE					PRIME COMPONENT SUPPLIER	COMPONENT MANUFACTURER			VOLATION		
07 S F		A	C O N T R L					N	B 1 3 5			N		
7	8	9	10	11	12					43	44			47

CAUSE DESCRIPTION

08	7	8	9	Just prior to finding the failed valve, tube leaks had been discovered in																																																																														80
09	7	8	9	the 2A LPCI heat exchanger. In order to perform the required maintenance, valve																																																																														80
10	7	8	9	2-1501-3A, the discharge valve on the 2A LPCI heat exchanger, had to be in the closed																																																																														80

(Continued)

FACILITY STATUS		% POWER	OTHER STATUS		METHOD OF DISCOVERY		DISCOVERY DESCRIPTION	
11 E		0 7 5	NA		B		NA	
7	8	9	10	12	13	44	45	46

FORM OF ACTIVITY RELEASED		CONTENT OF RELEASE	AMOUNT OF ACTIVITY		LOCATION OF RELEASE	
12 Z		Z	NA		NA	
7	8	9	10	11	44	45

PERSONNEL EXPOSURES

NUMBER	TYPE	DESCRIPTION
13 0 0 0	Z	NA
7	8	9

PERSONNEL INJURIES

NUMBER	DESCRIPTION
14 0 0 0	NA
7	8

OFFSITE CONSEQUENCES

15	7	8	9	NA																																																																														80
----	---	---	---	----	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	----

LOSS OR DAMAGE TO FACILITY

TYPE	DESCRIPTION
16 Z	NA
7	8

PUBLICITY

17	7	8	9	NA																																																																														80
----	---	---	---	----	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	----

ADDITIONAL FACTORS

18	7	8	9	NA																																																																														80
----	---	---	---	----	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	----

19	7	8	9	NA																																																																														80
----	---	---	---	----	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	--	----

NAME-

Desi Santanna

PHONE-

X-265

EVENT DISCRIPTION (CONTINUED)

valve last occurred in 1975. (50-237/1977-21)

CAUSE DESCRIPTION (Continued)

position. While manually closing the valve, the stem was bent. While performing maintenance on the valve stem, the feedback potentiometer (which is mechanically linked to the valve stem) inside the limitorque valve operator became misadjusted. Thus, the valve could not respond properly to demand signals from the remote valve position controller. Adjustment of the feedback potentiometer allowed the valve to function properly. The feedback potentiometer is a Model 5611 R1K(1)L.5 Heli-pot manufactured by Beckman Instruments, Inc.. The valves are 12"-300 PSI USA standard valve assemblies manufactured by Copes-Vulcan, a division of Blaw-Knox Company.

A request has been made to the Station Nuclear Engineering Department to review this event. Our current thinking is to increase the size of the stem on the 1501-3 valves and/or to install an appropriate isolation valve in the system for maintenance. We anticipate, however, that the final resolution of this problem might take approximately 18 months due to equipment delivery times. In the interim Caution tags will be attached to the 1501-3 valves to ensure appropriate care is taken when manually shutting the valves to isolate the system.

RECEIVED DOCUMENT
PROCESSING UNIT

1977 AUG 24 PM 12 22