

50-220

NRC DISTRIBUTION FOR PART 50 DOCKET MATERIAL

FILE NUMBER
INCIDENT REPORT

TO:

Mr. James P. O'Reilly

FROM:

Niagara Mohawk Power Corp.
Syracuse, New York
J. Bartlett

DATE OF DOCUMENT

5/10/77

DATE RECEIVED

5/16/77

☒ LETTER☐ NOTORIZED

PROP

INPUT FORM

NUMBER OF COPIES RECEIVED

☒ ORIGINAL
☐ COPY☒ UNCLASSIFIED

1516250

DESCRIPTION

Ltr. trans the following:

PLANT NAME:

Nine Mile Point Unit No. 1

(1-P)

RJL

ENCLOSURE

Licensee Event Report (RO 50-220/77-19 & 77-20)
on 4/27/77 concerning two containment oxygen
sampling system, check valves indicating leakage
in excess of the single valve criteria in Tech.
Specs.....

ACKNOWLEDGED

DO NOT REMOVE

NOTE: IF PERSONNEL EXPOSURE IS INVOLVED
SEND DIRECTLY TO KREGER/J. COLLINS

FOR ACTION/INFORMATION

BRANCH CHIEF:

W/3 CYS FOR ACTION

LIC. ASST.:

W/1 CYS

ACRS 16 CYS HOLDING/SENT

As CAT B

INTERNAL DISTRIBUTION

☒ REG FILE

NRC PDR

I & E (2)

MIPC

SCHROEDER/IPPOLITO

HOUSTON

NOVAK/CHECK

GRIMES

CASE

BUTLER

HANAUER

TEDESCO/MACCARY

EISENHUT

BAER

SHAO

VOLLMER/BUNCH

KREGER/J. COLLINS

EXTERNAL DISTRIBUTION

LPDR: Dsweso, NY

TIC:

NSIC:

CONTROL NUMBER

771360237

1941

1942

NIAGARA MOHAWK POWER CORPORATION

NIAGARA  MOHAWK300 ERIE BOULEVARD, WEST
SYRACUSE, N. Y. 13202

Regulatory

File CY-1

May 10, 1977

Mr. James P. O'Reilly
Director
United States Nuclear Regulatory Commission
Region I
631 Park Avenue
King of Prussia, PA. 19406



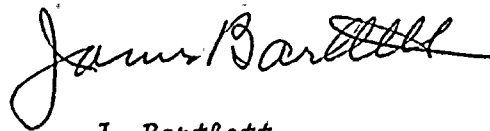
RE: Docket No. 50-220

Dear Mr. O'Reilly:

In accordance with Nine Mile Point Nuclear Station Unit #1 Technical Specification, we hereby submit Licensee Event Reports, LER 77-19 and LER 77-20.

These reports were completed in the format designated in the Licensee Event Report Instruction Booklet 00E-SS-001, dated October 2974, revised December 8, 1975.

Very truly yours,



J. Bartlett
Executive Vice President
for R.R. Schneider
Vice President -
Electric Production

MAS/mtm

Enc.

771360237

100-100000

100-100000

100-100000

LICENSEE EVENT REPORT

CONTROL BLOCK: 1 6

(PLEASE PRINT ALL REQUIRED INFORMATION)

LICENSEE NAME										LICENSE NUMBER										LICENSE TYPE					EVENT TYPE	
01	N	Y	N	M	P	1											4	1	1	1	1	0	1			
7	8	9				14	15										25	26			30	31	32			

CATEGORY		REPORT TYPE		REPORT SOURCE		DOCKET NUMBER					EVENT DATE				REPORT DATE										
01	CONT	-	-	T	L	0	5	0	-	0	2	2	0	0	4	2	7	7	7	0	5	1	0	7	7
7	8			57	58	59	60	61					68	69					74	75				80	

EVENT DESCRIPTION

02	During routine Type C isolation valve leak rate tests, two containment oxygen sampling																						
03	system check valves indicated leakage in excess of the single valve criteria in																						
04	Technical Specifications of 12.9 SCFH @ 22 psig. Leakages were 64.3 SCFH and 14.1																						
05	SCFH respectively.																						
06	LER 77-19, 77-20																						

SYSTEM CODE		CAUSE CODE		COMPONENT CODE					PRIME COMPONENT SUPPLIER		COMPONENT MANUFACTURER					VIOLATION	
07	S	D	E	V	A	L	V	E	X	A	X	9	9	9	N		
7	8	9	10	11	12				17	43	44			47	48		

CAUSE DESCRIPTION

08	"O" rings were replaced and the valves were retested to indicate leak rates of																						
09	.018 SCFH and .004 SCFH respectively.																						
10																							

FACILITY STATUS		% POWER			OTHER STATUS					METHOD OF DISCOVERY		DISCOVERY DESCRIPTION															
11	G	0	0	0	NA					B	NA																
7	8	9	10	12	13					44	45	46															

FORM OF ACTIVITY RELEASED		CONTENT OF RELEASE		AMOUNT OF ACTIVITY					LOCATION OF RELEASE																		
12	Z	Z	NA							NA																	
7	8	9	10	11					44	45																	

PERSONNEL EXPOSURES

NUMBER		TYPE		DESCRIPTION											
13	0	0	0	Z	NA										
7	8	9	11	12	13										

PERSONNEL INJURIES

NUMBER		DESCRIPTION												
14	0	0	0	NA										
7	8	9	11	12										

XXXXX CONSEQUENCES PROBABLE

15	NA																						
7	8	9																					

LOSS OR DAMAGE TO FACILITY

TYPE		DESCRIPTION											
16	Z	NA											
7	8	9	10										

PUBLICITY

17	NA																						
7	8	9																					

ADDITIONAL FACTORS

18	NA																						
7	8	9																					
19																							
7	8	9																					

NAME: _____ PHONE: _____

RECEIVED
U. S. N. R. C.
MAY 13 1977
OF PRUSSIA DT

1977 MAY 16 AM 11 51

RECEIVED DOCUMENT
PROCESSING UNIT

NAGARA POWER, POWER CORPORATION

TELECOPY MESSAGE

TO: 615-347-1151 (24) FROM: 615-343-2110 (1376) DATE/TIME 5/09/77
 Telephone Number Telephone Number

TO: James P. O'Reilly
 Director of Regulatory Operation
 USNRC Region I
 631 Park Avenue
 King of Prussia, PA. 19406

From: Niagara Mohawk Power Corporation
 Nine Mile Point Nuclear Station
 Unit #1
 P.O. Box #32
 Lycoming, New York 13093

SUBJECT: PROMPT REPORTABLE OCCURRENCE
 DOCKET NO. 50-220 LICENSE NO. DPR-63
 ASSIGNED LER NO. XXX 76-45

EVENT DATE: 12/76 REPORT DATE: 5/9/77

EVENT DESCRIPTION: During a review of chemical discharge records, it was discovered that the sodium limiting value for liquid effluents was exceeded (1.59 ppm vs limit of 1.5 ppm - see Environmental Technical Specifications 2.3-1)

COMPONENTS INVOLVED: None

CAUSE AND REMEDIAL ACTION: The surveillance procedure was revised to assure a more timely check on the concentration of discharges which have been made. A station modification to provide discharge throttling capability will prevent re-occurrence.

FACILITY STATUS:

2 THERMAL INT

c) Routine Startup _____
 d) Routine Shutdown _____
 e) Steady State Oper _____
 f) Load Change _____

g) Shutdown _____
 h) Refueling _____
 i) Other _____
 j) Not Applicable X

A written follow-up report will be sent within two weeks.

TELECOPY TO J.P. O'Reilly FROM T.J. Pez King DATE 5/09/77
 NAME NAME

TELECOPY MESSAGE

TO: 215-337-1550 (324) FROM: 315 342-2110 DATE/TIME 5/09/77
 Telephone Number Telephone Number

TO: James P. O'Reilly From: Niagara Mohawk Power Corporation
 Director of Regulatory Operation Nine Mile Point Nuclear Station
 USNRC Region I Unit #1
 631 Park Avenue P.O. Box #32
 King of Prussia, PA. 19406 Lycoming, New York 13093

SUBJECT: PROMPT REPORTABLE OCCURRENCE
 DOCKET NO. 50-220 LICENSE NO. DPR-63
 ASSIGNED LER NO. 77-21

EVENT DATE: 5/7/77 REPORT DATE: 5/9/77

EVENT DESCRIPTION: Failure to continuously monitor stack gross radioactivity as indicated in Environmental Technical Specifications 2.4.4.B.

COMPONENTS INVOLVED: Stack Sample Pump

CAUSE AND REMEDIAL ACTION: Stack sample pump was out of service for 9 hours and 20 minutes due to blown fuse. Fuse was replaced and pump was returned to service.

FACILITY STATUS: % THERMAL IN 0

c) Routine Startup	_____	g) Shutdown	X
d) Routine Shutdown	_____	h) Refueling	_____
e) Steady State Oper	_____	i) Other	_____
f) Load Change	_____	j) Not Applicable	_____

A written follow-up report will be sent within two weeks.

TELECOPY TO J P O'Reilly FROM T J Fackins DATE 5/09/77
 NAME NAME

