

30-220

NRC DISTRIBUTION FOR PART 50 DOCKET MATERIAL

FILE NUMBER
INCIDENT REPORT

TO: Mr. James P. O'Reilly

FROM: Niagara Mohawk Power Corp,
Syracuse N. Y. 13202
R. R. SchneiderDATE OF DOCUMENT
05-18-77DATE RECEIVED
05-27-77☒ LETTER
☒ ORIGINAL
☐ COPY☐ NOTORIZED
☒ UNCLASSIFIED

PROP

INPUT FORM

NUMBER OF COPIES RECEIVED

1 SIGNED

DESCRIPTION

1 page

ACKNOWLEDGED
DO NOT REMOVE

PLANT NAME: NINE MILE POINT UNIT # 1

jcm

ENCLOSURE

Licensee Event Report (R0-50-220/76-45)
on 12-XX-76 concerning sodium valve for liquid
effluents was 1.59 ppm vs a limit of 1.5 ppm.Licensee Event Report (R0-50-220/77-21)
on 05-07-77 concerning stack sample pump was out
of service for 9 hours 20 minutes, due to
blown fuse....

2 pages

NOTE: IF PERSONNEL EXPOSURE IS INVOLVED
SEND DIRECTLY TO KREGER/J. COLLINS

FOR ACTION/INFORMATION

BRANCH CHIEF:

W/3 CYS FOR ACTION

LIC. ASST.:

W/1 CYS

ACRS 16 CYS HOLDING/SENT

AS CAT B

INTERNAL DISTRIBUTION

REG FILE

NRC PDR

I & E (2)

MIPC

SCHROEDER/IPPOLITO

HOUSTON

NOVAK/CHECK

GRIMES

BUTLER

HANAUER

TEDESCO/MACCARY

EISENHUT

BAER

SHAO

VOLLMER/BUNCH

KREGER/J. COLLINS

EXTERNAL DISTRIBUTION

LPDR: 05/26/77, NY

TIC:

NSIC:

CONTROL NUMBER

771470172

RECEIVED
JUL 10 1963

NIAGARA MOHAWK POWER CORPORATION

NIAGARA  MOHAWK300 ERIE BOULEVARD, WEST
SYRACUSE, N. Y. 13202

Regulatory

File Cy.

May 18, 1977

Mr. James P. O'Reilly
Director
United States Nuclear Regulatory Commission
Region I
631 Park Avenue
King of Prussia, PA. 19406

RE: Docket No. 50-220

Dear Mr. O'Reilly:

In accordance with Nine Mile Point Nuclear Station Unit #1
Technical Specifications, we hereby submit Licensee Event Reports,
LER 76-45 and 77-21.

These reports were completed in the format designated in the
Licensee Event Report Instruction Booklet 00E-SS-001, dated
October 1974, revised December 8, 1975.

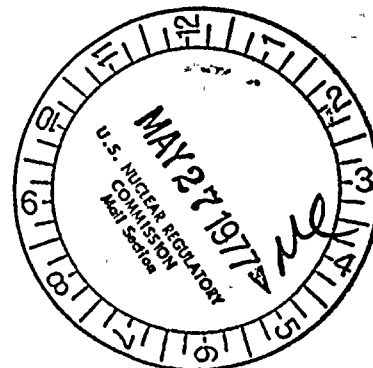
Very truly yours,



R.R. Schneider
Vice President -
Electric Production

MAS/mtm

Enc.



0 007

LICENSEE EVENT REPORT

CONTROL BLOCK:

[PLEASE PRINT ALL REQUIRED INFORMATION]

LICENSEE NAME														LICENSE NUMBER										LICENSE TYPE					EVENT TYPE	
01 N Y N M P 1														15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30										4 1 1 1 1					0 4	
7 8 9 10 11 12 13 14														15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30										26 27 28 29 30					31 32	

CATEGORY		REPORT TYPE	REPORT SOURCE	DOCKET NUMBER										EVENT DATE					REPORT DATE				
01 CON'T		-	T	0 5 0 - 0 2 2 0										1 2 x x 7 6					0 5 1 8 7 7				
57 58		59	60	61 62 63 64 65 66 67 68										69 70 71 72 73 74					75 76 77 78 79 80				

EVENT DESCRIPTION

02	7	8	9	During a review of chemical discharge records for December 1976, it was found that the																																																																													80
03	7	8	9	sodium valve for liquid effluents was 1.59 ppm vs a limit of 1.5 ppm. (ETS 2.3-1)																																																																													80
04	7	8	9	Pump out rate exceeded allowable for concentration in the make-up demineralizer waste																																																																													80
05	7	8	9	tank. Review of records not made until May 1977.																																																																													80
06	7	8	9	LER 76-45																																																																													80

SYSTEM CODE		CAUSE CODE		COMPONENT CODE					PRIME COMPONENT SUPPLIER		COMPONENT MANUFACTURER					VIOLATION	
07 W C		A		Z Z Z Z Z Z					Z		Z X X X					N	
7 8 9 10		11		12 13 14 15 16 17					43		44 45 46 47					48	

CAUSE DESCRIPTION

08	7	8	9	The surveillance procedure was revised to assure a more timely check on concentration of																																																																													80
09	7	8	9	of discharges which have been made. A station modification to provide discharge																																																																													80
10	7	8	9	throttling capability will prevent recurrence.																																																																													80

FACILITY STATUS		% POWER		OTHER STATUS		METHOD OF DISCOVERY		DISCOVERY DESCRIPTION									
11 E		0 9 7		NA		B		Routine surveillance									
7 8 9		10 11 12		13 14 15		44 45		46 47 48 49 50 51 52 53 54 55 56 57 58 59 60									

FORM OF ACTIVITY RELEASED		CONTENT OF RELEASE		AMOUNT OF ACTIVITY										LOCATION OF RELEASE									
12 Z		Z		NA										NA									
7 8 9		10 11		44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60																			

PERSONNEL EXPOSURES

NUMBER			TYPE	DESCRIPTION
13 0 0 0			Z	NA
7 8 9 10 11			12	13

PERSONNEL INJURIES

NUMBER			DESCRIPTION
14 0 0 0			NA
7 8 9 10 11			12

CONSEQUENCES PROBABLE

15	7	8	9	NA																																																																													80
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LOSS OR DAMAGE TO FACILITY

TYPE	DESCRIPTION
16 Z	NA
7 8 9	10

PUBLICITY

17	7	8	9	NA																																																																													80
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ADDITIONAL FACTORS

18	7	8	9	NA																																																																													80
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19	7	8	9																																																																														80
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LICENSEE EVENT REPORT

CONTROL BLOCK:

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1 6

[PLEASE PRINT ALL REQUIRED INFORMATION]

LICENSEE NAME					LICENSE NUMBER					LICENSE TYPE					EVENT TYPE	
0	1	N	Y	N			-			4	1	1	1	1	0	1
7	8	9		14	15				25	26				30	31	32

CATEGORY		REPORT TYPE	REPORT SOURCE	DOCKET NUMBER						EVENT DATE					REPORT DATE								
01	CON'T	-	T	0	5	0	-	0	2	2	0	0	5	0	7	7	7	0	5	1	8	7	7
7	8	57	58	59	60	61					68	69						75					80

EVENT DESCRIPTION

02	Stack sample pump was out of service for 9 hours 20 minutes due to blown fuse. Stack	80
03	gross radioactivity not monitored as required by ETS 2.4.4B. Technician replaced	80
04	fuse and restored service when performing weekly sample change. Reported 5/9/77..	80
05		80
06	LER 77-21	80

SYSTEM CODE		CAUSE CODE	COMPONENT CODE					COMPONENT SUPPLIER	COMPONENT MANUFACTURER				VIOLATION	
07	M C	E1	P	U	M	P	X	X	Z	Z	9	9	9	N
7	8	9	10						43	44				48

CAUSE DESCRIPTION

Operator did not recognize low sample flow alarm to initiate prompt restoration of service.

The high radiation sensors in the offgas line, main steam line, refueling platform, and reactor building ventilation performed gaseous effluent isolations. These were operable

FACILITY STATUS		% POWER			OTHER STATUS		METHOD OF DISCOVERY		DISCOVERY DESCRIPTION	
11	G	0	0	0	NA		B	Routine inspection		
7	8	9	10	11	12	13	44	45	46	80

FORM OF ACTIVITY RELEASED		CONTENT OF RELEASE		AMOUNT OF ACTIVITY		LOCATION OF RELEASE	
1	2	Z	Z	NA	NA		
7	8	9	10	11	44	45	80

PERSONNEL EXPOSURES

NUMBER				TYPE	DESCRIPTION
13	0	0	0	Z	NA

PERSONNEL INJURIES

NUMBER				DESCRIPTION	
1	4	0	0	0	NA

~~XXXXXX~~CONSEQUENCES PROBABLE

15	NA	80
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LOSS OR DAMAGE TO FACILITY

TYPE		DESCRIPTION
16	Z	NA

PUBLICITY

[illegible]

ADDITIONAL FACTORS

18 Release rate prior to incident of iodine and particulates $T_{1/2} > 8$ days was about
89 80
6 x 10⁻³% of limit and noble gas release < 50 pci/sec Unit Shutdown.
19 89 80

NAME: _____ PHONE: _____

RECEIVED DOCUMENT
PROCESSING UNIT

1911 MAY 26 PM 3 22

23 MAY 77 1:02

USNRC-REG. 1-