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DOCDATE: 07/21/77
DATE RCVD: 03/31/78

DOCTYPE: LETTER NOTARIZED: NO
SUBJECT:
LICENSEE EVENT REPT (RO 50-220/77-32) ON 07/01/77 CONCERNING DURING REFUELING, A WATER PHASE MATERIAL SAMPLE IN THE REACTOR VESSEL WAS NOT INSPECTED AS REQUIRED BY TECH SPEC 4.2.2.D...W/ATT LER 77-32 AND W/OUT LERS 77-33, 77-36 &77-37.

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July 21, 1977

REGULATORY DOCKET FILE COPY

RECEIVED U.S. NUCLEAR REGULATORY COMMISSION

Mr. James P. O'Reilly
Director
United States Nuclear Regulatory Commission
Region I
631 Park Avenue
King of Prussia, PA. 19406

RE: Docket No. 50-220

Dear Mr. O'Reilly:

In accordance with the Nine Mile Point Nuclear Station Unit #1 Technical Specifications, we hereby submit Licensee Event Reports, LER's 77-32, 77-33, 77-36 and 77-37.

These reports were completed in the format designated in the Licensee Event Report Instruction Booklet 00E-SS-001, dated October 1974, revised December 8, 1975.

Very truly yours,

ORIGINAL SIGNED BY R.R. SCHNEIDER

R.R. Schneider
Vice President -
Electric Production

MAS/mtm

Enclosure (3 copies)

cc: Director, I&E (40 copies)
Director, MIPC (3 copies)

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LICENSEE EVENT REPORT

CONTROL BLOCK: 0 1 8 2 1 9 8 (1–6) [PLEASE PRINT ALL REQUIRED INFORMATION]

	LICENSEE NAME	LICENSE NUMBER	LICENSE TYPE	EVENT TYPE
0 1	N Y N M P 1	0 0 – 0 0 0 0 0 – 0 0	4 1 1 1 1	0 3
(7–8)	(9–14)	(15–25)	(26–30)	(31–32)

	CATEGORY	REPORT TYPE	REPORT SOURCE	DOCKET NUMBER	EVENT DATE	REPORT DATE
0 1 CON'T	1 7	L	L	0 5 0 – 0 2 2 0	0 7 0 1 7 7	0 7 2 1 7 7
(7–8)	(57–58)	(59)	(60)	(61–68)	(69–74)	(75–80)

EVENT DESCRIPTION (LER 77-32). DURING REFUELING, A WATER PHASE
0 2 ~~SEE ATTACHMENT TO LER FOR DESCRIPTION.~~
0 3 MATERIAL SAMPLE IN THE REACTOR VESSEL WAS NOT INSPECTED
0 4 AS REQUIRED BY T.S. 4.2.2.D. OTHER INSPECTIONS AND ANAYSES
0 5 INDICATE THAT NO DETERIORATION OF THE SENSITIZED STAINLESS
0 6 STEEL HAS OCCURRED. ~~LER 77-32~~

	SYSTEM CODE	CAUSE CODE	COMPONENT CODE	PRIME COMPONENT SUPPLIER	COMPONENT MANUFACTURER	VIOLATION
0 7	C A	A	V E S S E L	N	G 0 8 0	Y
(7–9)	(10)	(11)	(12–17)	(43)	(44–47)	(48)

CAUSE DESCRIPTION INABILITY
0 8 ~~SEE ATTACHMENT~~ TO REMOVE AND COLLECT MATERIAL SAMPLE
0 9 PACKET WITH TOOLS CURRENTLY AVAILABLE. ATTEMPTS WILL BE MADE
1 0 FOR NEXT REFUELING TO DESIGN A TOOL, OR PLACE A NEW SAMPLE IN ACCESSIBLE LOCATION.

	FACILITY STATUS	% POWER	OTHER STATUS	METHOD OF DISCOVERY	DISCOVERY DESCRIPTION
1 1	H	0 0 0	NA	A	NA
(7–8)	(9)	(10–12)	(13–44)	(45)	(46–80)

	FORM OF ACTIVITY RELEASED	CONTENT OF RELEASE	AMOUNT OF ACTIVITY	LOCATION OF RELEASE
1 2	Z	Z	NA	NA
(7–8)	(9)	(10)	(11–44)	(45–80)

PERSONNEL EXPOSURES

	NUMBER	TYPE	DESCRIPTION
1 3	0 0 0	Z	NA
(7–8)	(9–11)	(12)	(13–80)

PERSONNEL INJURIES

	NUMBER	DESCRIPTION
1 4	0 0 0	NA
(7–8)	(9–11)	(12–80)

~~XXXXXX~~ CONSEQUENCES PROBABLE
1 5 NA

LOSS OR DAMAGE TO FACILITY

	TYPE	DESCRIPTION
1 6	Z	NA
(7–8)	(9)	(10–80)

PUBLICITY
1 7 NA

ADDITIONAL FACTORS
1 8 NA

1 9

NAME: P.E. Bole 7/27/77 (D) PHONE: 4927735

ATTACHMENT TO LER 77-32 for NMP #1

EVENT DESCRIPTION

During a routine refueling outage, a water phase material sample in the reactor vessel was not inspected as is required by Technical Specification 4.2.2.d. Inspections of the other material samples in steam and steam/water phase areas, and analysis of other conditions such as primary coolant chemistry indicate that no deterioration of the sensitized stainless steel has occurred. A Technical Specification change has been recommended to eliminate the requirement for a water phase material sample inspection for the current outage.

CAUSE DESCRIPTION

Due to the location and radioactivity of the material sample packet, it was not possible to remove and collect it with any tools currently available. During the next scheduled refueling outage, attempts will be made to design a tool for such a purpose, or a new material sample will be placed in the reactor in a location which permits retrieval.