

F 04/05/78

REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)
DISTRIBUTION FOR INCOMING MATERIAL

50-220

REC: OREILLY J P
NRC

ORG: SCHNEIDER R R
NIAGARA MOHAWK PWR

DOCDATE: 07/21/77
DATE RCVD: 03/31/78

DOCTYPE: LETTER NOTARIZED: NO
SUBJECT:

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LTR 1 ENCL 1

LICENSEE EVENT REPT (RO 50-220/77-32) ON 07/01/77 CONCERNING DURING
REFUELING, A WATER PHASE MATERIAL SAMPLE IN THE REACTOR VESSEL WAS NOT
INSPECTED AS REQUIRED BY TECH SPEC 4.2.2.D...W/ATT LER 77-32 AND W/OUT LERS
77-33, 77-36 & 77-37.

PLANT NAME: NINE MILE PT - UNIT 1

REVIEWER INITIAL: XJM
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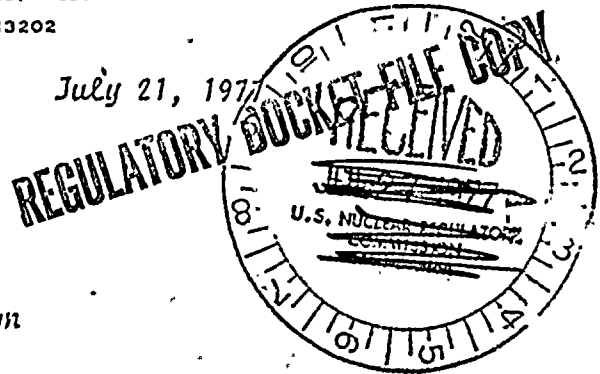
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*Boke*RECEIVED DISTRIBUTION SERVICES UNIT
NIAGARA MOHAWK POWER CORPORATIONNIAGARA  MOHAWK300 ERIE BOULEVARD, WEST
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1978 MAR 31 PM 1 16

US HRC
DISTRIBUTION SERVICES
BRANCH

July 21, 1977



Mr. James P. O'Reilly
Director
United States Nuclear Regulatory Commission
Region I
631 Park Avenue
King of Prussia, PA. 19406

RE: Docket No. 50-220

Dear Mr. O'Reilly:

In accordance with the Nine Mile Point Nuclear Station Unit #1 Technical Specifications, we hereby submit Licensee Event Reports, LER's 77-32, 77-33, 77-36 and 77-37.

These reports were completed in the format designated in the Licensee Event Report Instruction Booklet 00E-SS-001, dated October 1974, revised December 8, 1975.

Very truly yours,

ORIGINAL SIGNED BY R.R. SCHNEIDER

R.R. Schneider
Vice President -
Electric Production

MAS/mtm

Enclosure (3 copies)

cc: Director, I&E (40 copies)
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1977 JUL 27 AM 11 02

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NOTICE

ALL INFORMATION CONTAINED HEREIN IS UNCLASSIFIED

DATE 11/1/87 BY SP-10/BJ

LICENSEE EVENT REPORT

CONTROL BLOCK: 011812198

(PLEASE PRINT ALL REQUIRED INFORMATION)

LICENSEE NAME														LICENSE NUMBER												LICENSE TYPE						EVENT TYPE	
01	N	Y	N	M	P	I	0	0	-	0	0	0	0	-	0	0	4	1	1	1	1	0	3										
7	8	9				14	15									25	26					30	31	32									
01		CONT		CATEGORY		REPORT TYPE		REPORT SOURCE		DOCKET NUMBER								EVENT DATE				REPORT DATE											
01	57	58	L	L	0	5	0	-	0	2	2	0	0	7	0	1	7	7	0	7	2	2	7	7									
7	8					59	60	61					68	69					74	75				80									

EVENT DESCRIPTION - (LER 77-32). DURING REFUELING, A WATER PHASE
 SEE ATTACHMENT TO LER FOR DESCRIPTION.
 MATERIAL SAMPLE IN THE REACTOR VESSEL WAS NOT INSPECTED
 AS REQUIRED BY T.S. 4.2.2. D. OTHER INSPECTIONS AND ANALYSES
 INDICATE THAT NO DETERIORATION OF THE SENSITIZED STAINLESS
 STEEL HAS OCCURRED. LER 77-32

SYSTEM CODE				CAUSE CODE		COMPONENT CODE				PRIME COMPONENT SUPPLIER		COMPONENT MANUFACTURER				VIOLATION		
07	C	A		A		V	E	S	S	E	L	N		G	O	8	0	Y
7	8	9	10	11		12					17	43		44			47	48

CAUSE DESCRIPTION INABILITY
 SEE ATTACHMENT TO REMOVE AND COLLECT MATERIAL SAMPLE
 PACKET WITH TOOLS CURRENTLY AVAILABLE. ATTEMPTS WILL BE MADE
 FOR NEXT REFUELING TO DESIGN A TOOL, OR PLACE A NEW SAMPLE
 IN ACCESSIBLE LOCATION.

FACILITY STATUS		% POWER			OTHER STATUS			METHOD OF DISCOVERY		DISCOVERY DESCRIPTION		
11	H	0	0	0	NA			A		NA		
7	8	9	10	12	13			44	45	46		80

FORM OF ACTIVITY RELEASED		CONTENT OF RELEASE			AMOUNT OF ACTIVITY			LOCATION OF RELEASE		
12	Z	Z			NA			NA		
7	8	9	10	11				44	45	

PERSONNEL EXPOSURES					
NUMBER		TYPE		DESCRIPTION	
13	0	0	0	Z	NA
7	8	9	11	12	13

PERSONNEL INJURIES				
NUMBER		DESCRIPTION		
14	0	0	0	NA
7	8	9	11	12

CONSEQUENCES		PROBABLE	
15	NA		
7	8	9	

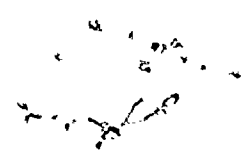
LOSS OR DAMAGE TO FACILITY			
TYPE		DESCRIPTION	
16	Z		NA
7	8	9	10

PUBLICITY			
17	NA		
7	8	9	

ADDITIONAL FACTORS			
18	NA		
7	8	9	

19			
7	8	9	

NAME: PEBore 7/27/77 (D) PHONE: 4927735



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11
1000

EVENT DESCRIPTION

During a routine refueling outage, a water phase material sample in the reactor vessel was not inspected as is required by Technical Specification 4.2.2.d. Inspections of the other material samples in steam and steam/water phase areas, and analysis of other conditions such as primary coolant chemistry indicate that no deterioration of the sensitized stainless steel has occurred. A Technical Specification change has been recommended to eliminate the requirement for a water phase material sample inspection for the current outage.

CAUSE DESCRIPTION

Due to the location and radioactivity of the material sample packet, it was not possible to remove and collect it with any tools currently available. During the next scheduled refueling outage, attempts will be made to design a tool for such a purpose, or a new material sample will be placed in the reactor in a location which permits retrieval.

