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SUBJECT:

LTR 1 ENCL 70

FORWARDING GE MARK II CONTAINMENT PROGRAM REPT ENTITLED: "FLUID -- STRUCTURE
INTERACTION ANALYSIS OF RAMSHEAD SAFETY RELIEF VALVE DISCHARGE IN MARK I STEEL
CONTAINMENT TORUS," NEDO-23834, JUNE 1978.

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NUCLEAR ENERGY

PROJECTS DIVISION

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MFN-315-78

July 28, 1978

U. S. Nuclear Regulatory Commission
Division of Project Management
Office of Nuclear Reactor Regulation
Washington, D.C. 20555

Attention: Mr. John F. Stolz (Chief)
Light Water Reactors Branch No. 1

Gentlemen:

SUBJECT: MARK II CONTAINMENT PROGRAM REPORT, "FLUID - STRUCTURE
INTERACTION ANALYSIS OF RAMSHEAD SAFETY RELIEF VALVE
DISCHARGE IN MARK I STEEL CONTAINMENT TORUS," NEDO-23834,
JUNE 1978

Reference: Letter with report, "Mark II Containment Program Report,
'Fluid - Structure Interaction Analysis of Ramshead
Safety Relief Valve Discharge in Mark I Steel Containment
Torus,' NEDO-23834, June 1978," to J. F. Stolz (NRC) from
L. H. Sobon (GE), dated July 7, 1978

Seventy (70) copies of the subject report (NEDO-23834) are being provided by General Electric Company on behalf of the Mark II Owners Group for your review, as part of the Mark II Containment Supporting Program, Task B.10. This report presents the results of an analysis of the fluid-structure interaction (FSI) effects on the major wall pressure data from safety relief valve (SRV) discharge. The studies conclude that FSI effects in the Mark I SRV tests were not significant and that this data can be used as a benchmark for the Mark II ramshead load prediction methodology.

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U.S. Nuclear Regulatory Commission

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July 28, 1978

The enclosed copies of NEDO-23834 are the printed versions of the ten (10) advance copies sent to you on July 7, 1978 (reference). Please discard or return the advance copies.

Sincerely,

L. J. Sobon for

L. J. Sobon, Manager
BWR Containment Licensing
Containment Improvement Programs

LJs:bp/1290

Enclosures (70)

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