

CATEGORY 1

REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

ACCESSION NBR: 9610160021 DOC. DATE: 96/10/08 NOTARIZED: NO DOCKET # 05000400
 FACIL: 50-400 Shearon Harris Nuclear Power Plant, Unit 1, Carolina
 AUTH. NAME AUTHOR AFFILIATION
 ROBINSON, W.R. Carolina Power & Light Co.
 RECIPIENT NAME RECIPIENT AFFILIATION
 Document Control Branch (Document Control Desk)

SUBJECT: Proposes implementing change to Bases for TSs 3/4.6.1.4 & 3/4.6.1.6 to change calculated peak pressure for MSLB event.

DISTRIBUTION CODE: A001D COPIES RECEIVED: LTR 1 ENCL 1 SIZE: 2 + 2
 TITLE: OR Submittal: General Distribution

NOTES: Application for permit renewal filed. 05000400

RECIPIENT ID CODE/NAME	COPIES	LTTR	ENCL	RECIPIENT ID CODE/NAME	COPIES	LTTR	ENCL
PD2-1 LA	1		1	PD2-1 PD	1		1
LE, N	1		1				
INTERNAL: ACRS	1		1	<u>FILE CENTER</u> 01	1		1
NRR/DE/EMCB	1		1	NRR/DRCH/HICB	1		1
NRR/DSSA/SPLB	1		1	NRR/DSSA/SRXB	1		1
NUDOCS-ABSTRACT	1		1	OGC/HDS2	1		0
EXTERNAL: NOAC	1		1	NRC PDR	1		1

C
A
T
E
G
O
R
Y

1

D
O
C
U
M
E
N
T

NOTE TO ALL "RIDS" RECIPIENTS:
 PLEASE HELP US TO REDUCE WASTE. TO HAVE YOUR NAME OR ORGANIZATION REMOVED FROM DISTRIBUTION LISTS
 OR REDUCE THE NUMBER OF COPIES RECEIVED BY YOU OR YOUR ORGANIZATION, CONTACT THE DOCUMENT CONTROL
 DESK (DCD) ON EXTENSION 415-2083

TOTAL NUMBER OF COPIES REQUIRED: LTTR 13 ENCL 12



Carolina Power & Light Company
PO Box 165
New Hill NC 27562

William R. Robinson
Vice President
Harris Nuclear Plant

OCT 8 1996

SERIAL: HNP-96-064

United States Nuclear Regulatory Commission
ATTENTION: Document Control Desk
Washington, DC 20555

SHEARON HARRIS NUCLEAR POWER PLANT
DOCKET NO. 50-400/LICENSE NO. NPF-63
TECHNICAL SPECIFICATION BASES CHANGE - CONTAINMENT ANALYSIS

Gentlemen:

In accordance with 10 CFR 50.36(a), the Harris Nuclear Plant is proposing to implement a change to the Bases for Technical Specifications (TS) 3/4.6.1.4 and 3/4.6.1.6.

In response to NRC Information Notice 87-65, "Plant Operation Beyond Analyzed Conditions," an analysis was performed to evaluate the temperature distribution inside containment. That analysis concluded that the containment temperature is normally stratified, and under worst case conditions, would result in an average temperature of 135°F if thoroughly mixed. Containment temperature and pressure for the limiting MSLB and LOCA scenarios were recalculated using 135°F as the pre-accident temperature. 120°F had been used as the pre-accident temperature in previous analyses.

The revised analyses show that the peak temperature and pressure values remain within the accident acceptance criteria. The peak containment pressure following the most limiting LOCA scenario is essentially unchanged and remains below the peak accident pressure, Pa, as stated in TS 3.6.1.2 and the corresponding Bases. Therefore, offsite dose projections are not affected and there will be no increase in the consequences of this scenario.

However, the change in calculated peak pressure for the MSLB event will require revision of the Bases for Internal Pressure (TS 3.6.1.4) and Containment Structural Integrity (TS 3.6.1.6). Specifically, the peak pressure expected from a postulated MSLB event is now 41.2 psig, which is an increase above the current Bases peak values of 40.9 psig (TS 3.6.1.4) and 41 psig (TS 3.6.1.6). However, the revised peak pressure of 41.2 psig remains below the maximum internal design pressure of 45 psig, as stated in TS 5.2.2, Design Pressure and Temperature.

1/1
ADD

9610160021 961008
PDR ADOCK 05000400
P PDR

150081

Document Control Desk
HNP-96-064/ Page 2

The attached retyped Bases pages B 3/4 6-1 and B 3/4 6-2 are provided for your information.

Questions regarding this matter may be referred to Mr. T. D. Walt at (919) 362-2711.

Sincerely,

A handwritten signature in cursive script, appearing to read "J. S. Robinson".

Enclosure

LSR/lsr

c: Mr. J. B. Brady-NRC Resident Inspector
Mr. S. D. Ebnetter-NRC Regional Administrator
Mr. N. B. Le-NRC Project Manager