

		GENERAL EMERGENCY							SITE AREA EMERGENCY							ALERT							UNUSUAL EVENT																																																															
		Release of gaseous radioactivity resulting in offsite dose greater than 1000 mrem TEDE or 5000 mrem thyroid CDE [pg. 35]							Release of gaseous radioactivity resulting in offsite dose greater than 100 mrem TEDE or 500 mrem thyroid CDE [pg. 37]							Release of gaseous or liquid radioactivity resulting in offsite dose greater than 10 mrem TEDE or 50 mrem thyroid CDE [pg. 27]							Release of gaseous or liquid radioactivity greater than 2 times the ODAH limits for 60 minutes or longer [pg. 23]																																																															
		1	2	3	4	5	DEF	1	2	3	4	5	DEF	1	2	3	4	5	DEF	1	2	3	4	5	DEF																																																													
R Abnormal Rad Levels / Rad Effluent	1 Rad Effluent	RG1.1 Reading on ANY Table R-1 effluent radiation monitor greater than column "GE" for 15 minutes or longer. RG1.2 Dose assessment using actual meteorology indicates doses greater than 1000 mrem TEDE or 5000 mrem thyroid CDE at or beyond the SITE BOUNDARY. [Preferred] RG1.3 Field survey results indicate EITHER of the following at or beyond the SITE BOUNDARY: • Closed window dose rates greater than 1000 mR/hr expected to continue for greater than or equal to 60 min. • Analyses of field survey samples indicate thyroid CDE greater than 5000 mrem for 60 min. of inhalation.							RS1.1 Reading on ANY Table R-1 effluent radiation monitor greater than column "SAE" for 15 minutes or longer. RS1.2 Dose assessment using actual meteorology indicates doses greater than 100 mrem TEDE or 500 mrem thyroid CDE at or beyond the SITE BOUNDARY. [Preferred] RS1.3 Field survey results indicate EITHER of the following at or beyond the SITE BOUNDARY: • Closed window dose rates greater than 100 mR/hr expected to continue for greater than or equal to 60 min. • Analyses of field survey samples indicate thyroid CDE greater than 500 mrem for 60 min. of inhalation.							RA1.1 Reading on ANY Table R-1 effluent radiation monitor greater than column "ALERT" for 15 minutes or longer. RA1.2 Dose assessment using actual meteorology indicates doses greater than 10 mrem TEDE or 50 mrem thyroid CDE at or beyond the SITE BOUNDARY. [Preferred] RA1.3 Analysis of a liquid effluent sample indicates a concentration or release rate that would result in doses greater than 10 mrem TEDE or 50 mrem thyroid CDE at or beyond the SITE BOUNDARY for one hour of exposure. RA1.4 Field survey results indicate EITHER of the following at or beyond the SITE BOUNDARY: • Closed window dose rates greater than 10 mR/hr expected to continue for 60 minutes or longer. • Analyses of field survey samples indicate thyroid CDE greater than 50 mrem for one hour of inhalation.							RU1.1 Reading on ANY Table R-1 effluent radiation monitor greater than column "NOUE" for 60 minutes or longer. RU1.2 Reading on ANY effluent radiation monitor greater than 2 times the alarm setpoint established by a current radioactivity discharge permit for 60 minutes or longer. RU1.3 Sample analyses for a gaseous or liquid release indicates a concentration or release rate greater than 2 times the ODAH limits for 60 minutes or longer.																																																															
	2 Irradiated Fuel Event	Spent fuel pool level cannot be restored to at least the top of the fuel racks for 60 minutes or longer [pg. 37]							Spent fuel pool level at the top of the fuel racks [pg. 34]							Significant lowering of water level above, or damage to, irradiated fuel [pg. 29]							Unplanned loss of water level above irradiated fuel [pg. 25]																																																															
	3 Area Radiation Levels	RG2.1 Spent fuel pool level cannot be restored to at least 16.36 feet for 60 minutes or longer.							RS2.1 Lowering of spent fuel pool level to 16.36 feet.							RA2.1 Uncertainty of irradiated fuel in the REFUELING PATHWAY RA2.2 Damage to irradiated fuel resulting in a release of radioactivity from the fuel as indicated by a HI Rad alarm for ANY of the following ARMS: • Spent Fuel Pool Area, RI-9178 • North Refuel Floor, RI-9163 • New Fuel Vault Area, RI-9163 • South Refuel Floor, RI-9164 OR Reading greater than 5 R/hr on ANY of the following radiation monitors (in Mode 5 only): • NW Drywell Area HI Range Rad Monitor, RIM-9184A • South Drywell Area HI Range Rad Monitor, RIM-9184B RA2.3 Lowering of spent fuel pool level to 25.17 feet. Radiation levels that impede access to areas necessary for normal plant operation. [pg. 31] RA3.1 Dose rate greater than 15 mR/hr in ANY of the following areas: • Control Room (RM-9162) • Central Alarm Station (by survey)							RU2.1 a. UNPLANNED water level drop in the REFUELING PATHWAY as indicated by ANY of the following: • Report to control room (visual observation) • Fuel pool level indication (LI-3413) less than 36 feet and lowering • WR GEMAC Floodup indication (LI-4541) coming on scale AND b. UNPLANNED rise in area radiation levels as indicated by ANY of the following radiation monitors: • Spent Fuel Pool Area, RI-9178 • North Refuel Floor, RI-9163 • New Fuel Vault Area, RI-9163 • South Refuel Floor, RI-9164 • NW Drywell Area HI Range Rad Monitor, RIM-9184A • South Drywell Area HI Range Rad Monitor, RIM-9184B																																																															
		<table><tr><th colspan="14">Table R-1 - Effluent Monitor Classification Thresholds</th></tr><tr><th colspan="2">Monitor</th><th>GE</th><th>SAE</th><th>Alert</th><th>NOUE</th></tr><tr><td rowspan="4">Gaseous</td><td>Reactor Building ventilation rad monitor (Kaman 3/4, 5/6, 7/8)</td><td>1.0E+00 uCi/cc</td><td>1.0E-01 uCi/cc</td><td>1.0E-02 uCi/cc</td><td>1.0E-03 uCi/cc</td></tr><tr><td>Turbine Building ventilation rad monitor (Kaman 1/2)</td><td>1.0E+00 uCi/cc</td><td>1.0E-01 uCi/cc</td><td>1.0E-02 uCi/cc</td><td>1.0E-03 uCi/cc</td></tr><tr><td>Offgas Stack rad monitor (Kaman 9/20)</td><td>4.5E+03 uCi/cc</td><td>4.5E+02 uCi/cc</td><td>4.5E+01 uCi/cc</td><td>2.0E-01 uCi/cc</td></tr><tr><td>UHFSS rad monitor (Kaman 12)</td><td>---</td><td>1.0E-01 uCi/cc</td><td>1.0E-02 uCi/cc</td><td>1.0E-03 uCi/cc</td></tr><tr><td rowspan="3">Liquid</td><td>GSW rad monitor (RIS-4767)</td><td>---</td><td>---</td><td>2.0E+04 cps</td><td>2.0E+03 cps</td></tr><tr><td>RHSW & ESW rad monitor (RM-1997)</td><td>---</td><td>---</td><td>1.0E+04 cps</td><td>8.0E+02 cps</td></tr><tr><td>RHSW & ESW Rupture Disc rad monitor (RM-4268)</td><td>---</td><td>---</td><td>2.0E+04 cps</td><td>1.0E+03 cps</td></tr></table>																												Table R-1 - Effluent Monitor Classification Thresholds														Monitor		GE	SAE	Alert	NOUE	Gaseous	Reactor Building ventilation rad monitor (Kaman 3/4, 5/6, 7/8)	1.0E+00 uCi/cc	1.0E-01 uCi/cc	1.0E-02 uCi/cc	1.0E-03 uCi/cc	Turbine Building ventilation rad monitor (Kaman 1/2)	1.0E+00 uCi/cc	1.0E-01 uCi/cc	1.0E-02 uCi/cc	1.0E-03 uCi/cc	Offgas Stack rad monitor (Kaman 9/20)	4.5E+03 uCi/cc	4.5E+02 uCi/cc	4.5E+01 uCi/cc	2.0E-01 uCi/cc	UHFSS rad monitor (Kaman 12)	---	1.0E-01 uCi/cc	1.0E-02 uCi/cc	1.0E-03 uCi/cc	Liquid	GSW rad monitor (RIS-4767)	---	---	2.0E+04 cps	2.0E+03 cps	RHSW & ESW rad monitor (RM-1997)	---	---	1.0E+04 cps	8.0E+02 cps	RHSW & ESW Rupture Disc rad monitor (RM-4268)	---	---	2.0E+04 cps	1.0E+03 cps
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E ISFSI	1 Confinement Boundary	<table><tr><th colspan="14">Table E-1 Cask On-Contact Dose Rates</th></tr><tr><th colspan="14">61BT DSC</th></tr><tr><td colspan="14">3 feet from HSM Surface 800 mrem/hr Outside HSM Door - CL or DSC 200 mrem/hr End Shield Wall Exterior 40 mrem/hr</td></tr></table> Damage to a loaded cask CONFINEMENT BOUNDARY [pg. 61] E-HU1.1 Damage to a loaded cask CONFINEMENT BOUNDARY as indicated by an on-contact radiation reading greater than the values shown on Table E-1 on the surface of the spent fuel cask.																												Table E-1 Cask On-Contact Dose Rates														61BT DSC														3 feet from HSM Surface 800 mrem/hr Outside HSM Door - CL or DSC 200 mrem/hr End Shield Wall Exterior 40 mrem/hr																												
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3 feet from HSM Surface 800 mrem/hr Outside HSM Door - CL or DSC 200 mrem/hr End Shield Wall Exterior 40 mrem/hr																																																																																						
1 Security	HOSTILE ACTION resulting in loss of physical control of the facility [pg. 95]							HOSTILE ACTION within the PROTECTED AREA [pg. 92]							HOSTILE ACTION within the OWNER CONTROLLED AREA or airborne attack threat within 30 minutes [pg. 85]							Confirmed SECURITY CONDITION or threat [pg. 78]																																																																
	1	2	3	4	5	DEF	1	2	3	4	5	DEF	1	2	3	4	5	DEF	1	2	3	4	5	DEF																																																														
	HG1.1 A HOSTILE ACTION is occurring or has occurred within the PROTECTED AREA as reported by the DAEC Security Shift Supervisor. AND EITHER of the following has occurred: 1. ANY of the following safety functions cannot be controlled or maintained: • Reactivity control • RPV water level • RCS heat removal OR 2. Damage to spent fuel has occurred or is IMMINENT							HS1.1 A HOSTILE ACTION is occurring or has occurred within the PROTECTED AREA as reported by the DAEC Security Shift Supervisor. HA1.1 A HOSTILE ACTION is occurring or has occurred within the OWNER CONTROLLED AREA as reported by the DAEC Security Shift Supervisor. HA1.2 A validated notification from NRC of an aircraft attack threat within 30 minutes of the site.							HU1.1 Damage to a loaded cask CONFINEMENT BOUNDARY as indicated by an on-contact radiation reading greater than the values shown on Table E-1 on the surface of the spent fuel cask. HU1.2 Damage to a loaded cask CONFINEMENT BOUNDARY as indicated by an on-contact radiation reading greater than the values shown on Table E-1 on the surface of the spent fuel cask. HU1.3 Confirmed SECURITY CONDITION or threat [pg. 78]							HU1.1 Damage to a loaded cask CONFINEMENT BOUNDARY as indicated by an on-contact radiation reading greater than the values shown on Table E-1 on the surface of the spent fuel cask. HU1.2 Damage to a loaded cask CONFINEMENT BOUNDARY as indicated by an on-contact radiation reading greater than the values shown on Table E-1 on the surface of the spent fuel cask. HU1.3 Confirmed SECURITY CONDITION or threat [pg. 78]																																																																
2 Seismic Event																						Seismic event greater than OBE level [pg. 60]																																																																
3 Natural or Tech. Hazard																						Hazardous event [pg. 81]																																																																
	1	2	3	4	5	DEF	1	2	3	4	5	DEF	1	2	3	4	5	DEF	1	2	3	4	5	DEF																																																														
																						HU3.1 A tornado strike within the PROTECTED AREA HU3.2 Internal room or area flooding of a magnitude sufficient to require manual or automatic electrical isolation of a SAFETY SYSTEM component needed for the current operating mode. HU3.3 Movement of personnel within the PROTECTED AREA is impeded due to an offsite event involving hazardous materials (e.g., an offsite chemical spill or toxic gas release). HU3.4 A hazardous event that results in on-site conditions sufficient to prohibit the plant staff from accessing the site via personal vehicles. HU3.5 River level above 757 feet. HU3.6 River Water Supply (RWS) at low level alarm.																																																																
H Hazards	4 Fire																						FIRE potentially degrading the level of safety of the plant [pg. 83]																																																															
	1	2	3	4	5	DEF	1	2	3	4	5	DEF	1	2	3	4	5	DEF	1	2	3	4	5	DEF																																																														
																						HU4.1 A FIRE is not extinguished within 15 minutes of ANY of the following FIRE detection indicators: • Report from the field (i.e., visual observation) • Receipt of multiple (more than 1) fire alarms or indications • Field verification of a single fire alarm AND The FIRE is located within ANY Table H-1 plant rooms or areas HU4.2 Receipt of a single fire alarm with no other indications of a FIRE AND The FIRE is located within ANY Table H-1 plant rooms or areas AND The existence of a FIRE is not verified within 30 minutes of alarm receipt. HU4.3 A FIRE within the plant or ISFSI PROTECTED AREA not extinguished within 60 minutes of the initial report, alarm or indication. HU4.4 A FIRE within the plant or ISFSI PROTECTED AREA that requires firefighting support by an offsite fire response agency to extinguish.																																																																
5 Control Room Evacuation	Inability to control a key safety function from outside the Control Room [pg. 84]							Control Room evacuation resulting in transfer of plant control to alternate location [pg. 90]							Internal room or area flooding of a magnitude sufficient to require manual or automatic electrical isolation of a SAFETY SYSTEM component needed for the current operating mode.																																																																							
	1	2	3	4	5	DEF	1	2	3	4	5	DEF	1	2	3	4	5	DEF	1	2	3	4	5	DEF																																																														
	HS5.1 An event has resulted in plant control being transferred from the Control Room to the Remote Shutdown Panel (1C388). AND Control of ANY of the following key safety functions is not reestablished within 20 minutes: • Reactivity control • RPV water level • RCS heat removal							HA5.1 An event has resulted in plant control being transferred from the Control Room to the Remote Shutdown Panel (1C388).							HU5.1 An event has resulted in plant control being transferred from the Control Room to the Remote Shutdown Panel (1C388).							HU5.1 An event has resulted in plant control being transferred from the Control Room to the Remote Shutdown Panel (1C388).																																																																
6 ED Judgment	Other conditions existing that in the judgment of the Emergency Coordinator warrant declaration of General Emergency [pg. 38]							Other conditions existing that in the judgment of the Emergency Coordinator warrant declaration of Site Area Emergency [pg. 95]							Other conditions existing that in the judgment of the Emergency Coordinator warrant declaration of an Alert [pg. 41]							Other conditions existing that in the judgment of the Emergency Coordinator warrant declaration of a RE [pg. 27]																																																																
	1	2	3	4	5	DEF	1	2	3	4	5	DEF	1	2	3	4	5	DEF	1	2	3	4	5	DEF																																																														
	HG6.1 Other conditions exist which in the judgment of the Emergency Director indicate that events are in progress or have occurred which involve actual or IMMINENT substantial core degradation or melting with potential for loss of containment integrity or HOSTILE ACTION that results in an actual loss of physical control of the facility. Releases can be reasonably expected to exceed EPA Protective Action Guideline exposure levels offsite for more than the immediate site area.							HS6.1 Other conditions exist which in the judgment of the Emergency Director indicate that events are in progress or have occurred which involve actual or likely major failures of plant functions needed for protection of the public or HOSTILE ACTION that results in intentional damage or malicious acts. (1) toward site personnel or equipment that could lead to the likely failure of, or (2) that prevent effective access to equipment needed for the protection of the public. Any releases are not expected to result in exposure levels which exceed EPA Protective Action Guideline exposure levels beyond the site boundary.							HA6.1 Other conditions exist which, in the judgment of the Emergency Director, indicate that events are in progress or have occurred which involve an actual or potential substantial degradation of the level of safety of the plant or a security event that involves probable life threatening risk to site personnel or damage to site equipment because of HOSTILE ACTION. Any releases are expected to be limited to small fractions of the EPA Protective Action Guideline exposure levels.							HU6.1 Other conditions exist which in the judgment of the Emergency Director indicate that events are in progress or have occurred which indicate a potential degradation of the level of safety of the plant or indicate a security threat to facility protection has been initiated. No releases of radioactive material requiring offsite response or monitoring are expected unless further degradation of SAFETY SYSTEMS occurs.																																																																

GENERAL EMERGENCY										SITE AREA EMERGENCY										ALERT										UNUSUAL EVENT										
1 Loss of Emergency AC Power	Prolonged loss of ALL offsite and ALL onsite AC power to essential buses [pg. 120]										Loss of ALL offsite and ALL onsite AC power to essential buses for 15 minutes or longer [pg. 117]										Loss of ALL but one AC power source to essential buses for 15 minutes or longer [pg. 115]										Loss of ALL offsite AC power capability to essential buses for 15 minutes or longer [pg. 100]									
	1 2 3 4 5 6 7 8 9 10										1 2 3 4 5 6 7 8 9 10										1 2 3 4 5 6 7 8 9 10										1 2 3 4 5 6 7 8 9 10									
	c. S01.1 a. Loss of ALL offsite and ALL onsite AC power to 1A3 and 1A4. AND b. EITHER: • Restoration of at least one AC essential bus in less than 4 hours is not likely. OR • RPV level cannot be restored and maintained above .25 inches										c. SS1.1 Loss of ALL offsite and ALL onsite AC power to 1A3 and 1A4 for 15 minutes or longer.										c. SA1.1 a. AC power capability to 1A3 and 1A3 is reduced to a single power source for 15 minutes or longer. AND b. ANY additional single power source failure will result in a loss of ALL AC power to SAFETY SYSTEMS.										c. SU1.1 Loss of ALL offsite AC power capability to 1A3 AND 1A4 for 15 minutes or longer.									
2 Loss of Vital DC Power	Loss of all AC and Vital DC power sources for 15 minutes or longer [pg. 122]										Loss of all Vital DC power for 15 minutes or longer [pg. 115]																													
	1 2 3 4 5 6 7 8 9 10										1 2 3 4 5 6 7 8 9 10																													
c. SG2.1 a. Loss of ALL offsite and ALL onsite AC power to 1A3 and 1A4 for 15 minutes or longer. AND b. Indicated voltage is less than 105 VDC on BOTH Div 1 and Div 2 125 VDC buses for 15 minutes or longer.										c. SS2.1 Indicated voltage is less than 105 VDC on BOTH Div 1 and Div 2 125 VDC buses for 15 minutes or longer.																														
3 Loss of Control Room Indications																					UNPLANNED loss of Control Room indications for 15 minutes or longer with a significant transient in progress [pg. 111]										UNPLANNED loss of Control Room indications for 15 minutes or longer [pg. 103]									
																					1 2 3 4 5 6 7 8 9 10										1 2 3 4 5 6 7 8 9 10									
c. SA3.1 a. An UNPLANNED event results in the inability to monitor one or more Table S-1 parameters from within the Control Room for 15 minutes or longer AND b. Any of the Table S-2 transient events are in progress																														c. SU3.1 An UNPLANNED event results in the inability to monitor one or more of the Table S-1 parameters from within the Control Room for 15 minutes or longer.										
4 RCS Activity											Table S-2: Significant Transients • Automatic or manual rubback greater than 25% thermal reactor power • Electrical load rejection greater than 25% full electrical load • Reactor scram • ECCS actuation • Thermal power oscillations greater than 10%										Table S-1: Safety System Parameters • Reactor power • RPV Water Level • RPV Pressure • Primary Containment Pressure • Suppression Pool Level • Suppression Pool Temperature										Reactor coolant activity greater than Technical Specification allowable limits [pg. 103]									
																															c. SU4.1 Pre-treatment Offgas System (RM-4104) Hi-Hi Radiation Alarm. c. SU4.2 Sample analysis indicates that reactor coolant specific activity is greater than 2.0 µCi/gm dose equivalent I-131 for 12 hours or longer									
5 RCS Leakage																					RCS leakage for 15 minutes or longer [pg. 104]																			
																					1 2 3 4 5 6 7 8 9 10										1 2 3 4 5 6 7 8 9 10									
S System Malfunc.																					Automatic or manual trip fails to shut down the reactor [pg. 106]																			
																					1 2 3 4 5 6 7 8 9 10										1 2 3 4 5 6 7 8 9 10									
6 RPS Failure	Inability to shut down the reactor causing a challenge to core cooling or RCS heat removal [pg. 119]										Automatic or manual scram fails to shut down the reactor and subsequent manual actions taken at the reactor control consoles are not successful in shutting down the reactor [pg. 113]										Automatic or manual trip fails to shut down the reactor [pg. 106]																			
	1 2 3 4 5 6 7 8 9 10										1 2 3 4 5 6 7 8 9 10										1 2 3 4 5 6 7 8 9 10										1 2 3 4 5 6 7 8 9 10									
c. SS6.1 a. An automatic or manual scram did not shutdown the reactor. AND b. ALL of the following manual actions taken at 1C05 are not successful in lowering reactor power below 5% power: • Manual Scram Pushbuttons • Mode Switch to Shutdown • Alternate Rod Insertion (ARI) AND c. EITHER of the following conditions exist: • RPV level cannot be restored and maintained above .25 inches. OR • HCL (Graph 4 of EOP 2) exceeded										c. SA6.1 a. An automatic or manual scram did not shutdown the reactor. AND b. ALL of the following manual actions taken at 1C05 are not successful in lowering reactor power below 5% power: • Manual Scram Pushbuttons • Mode Switch to Shutdown • Alternate Rod Insertion (ARI)										c. SU6.1 a. An automatic scram did not shutdown the reactor. AND b. ANY of the following manual actions taken at 1C05 are successful in lowering reactor power below 5% power: • Manual Scram Pushbuttons • Mode Switch to Shutdown • Alternate Rod Insertion (ARI) c. SU6.2 a. A manual scram did not shutdown the reactor. AND b. EITHER of the following: 1. ANY of the following manual actions taken at 1C05 are successful in lowering reactor power below 5% power: • Manual Scram Pushbuttons • Mode Switch to Shutdown • Alternate Rod Insertion (ARI) OR 2. A subsequent automatic scram is successful in shutting down the reactor.																				
7 Loss of Comm.																					Loss of all onsite or offsite communications capabilities [pg. 108]																			
																					1 2 3 4 5 6 7 8 9 10										1 2 3 4 5 6 7 8 9 10									
c. SU7.1 Loss of ALL of the following onsite communication methods: • Plant Operations Radio System • In-Plant Phone System • Plant Paging System (Gaitronics) c. SU7.2 Loss of ALL of the following offsite response organization communications methods: • DAEC Air-Cell Phone • All telephone lines (PBX and commercial) • Cell Phones (including fixed cell phone system) • Control Room fixed satellite phone system • FTS Phone system c. SU7.3 Loss of ALL of the following NRC communications methods: • FTS Phone system • All telephone lines (PBX and commercial) • Cell Phones (including fixed cell phone system) • Control Room fixed satellite phone system																																								
8 Hazardous Event Affecting Safety Systems	Table S-3 Hazardous Events • Seismic event (earthquake) • Internal or external flooding event • High winds or tornado strike • FIRE • EXPLOSION • River level above 757 feet • River Water Supply (RWS) pit low level alarm • Other events with similar hazard characteristics as determined by the Shift Manager or Emergency Director										Hazardous event affecting a SAFETY SYSTEM needed for the current operating mode [pg. 115]																													
	1 2 3 4 5 6 7 8 9 10										1 2 3 4 5 6 7 8 9 10										1 2 3 4 5 6 7 8 9 10										1 2 3 4 5 6 7 8 9 10									
c. SA8.1 a. The occurrence of ANY of the Table S-3 hazardous events: AND b. 1. Event damage has caused indications of degraded performance in one train of a SAFETY SYSTEM needed for the current operating mode. AND 2. EITHER of the following: • Event damage has caused indications of degraded performance to a second train of the SAFETY SYSTEM needed for the current operating mode. OR • The event has resulted in VISIBLE DAMAGE to the second train of a SAFETY SYSTEM needed for the current operating mode. AND c. Loss of the safety function of a single train SAFETY SYSTEM.																																								
F Fission Product Barrier Degradation	FG1										FS1										FA1																			
	1 2 3 4 5 6 7 8 9 10										1 2 3 4 5 6 7 8 9 10										1 2 3 4 5 6 7 8 9 10										1 2 3 4 5 6 7 8 9 10									
Loss of any two barriers and Loss or Potential Loss of third barrier (Table F-1) [pg. 62]										Loss or potential loss of any two barriers (Table F-1) [pg. 62]										Any Loss or any Potential Loss of either Fuel Clad or RCS barrier (Table F-1) [pg. 62]																				
Table F-1 Fission Product Barrier Matrix																																								
Category	Fuel Clad (FC) Barrier [pg. 66]										Reactor Coolant System (RCS) Barrier [pg. 69]										Containment (CNTMT) Barrier [pg. 72]																			
	□ Loss □ Potential Loss										□ Loss □ Potential Loss										□ Loss □ Potential Loss																			
1 Primary Containment Conditions	Not Applicable										Not Applicable										Not Applicable										Not Applicable									
	Not Applicable										Not Applicable										Not Applicable										Not Applicable									
2 RPV Water Level	c. A. SAG entry is required.										c. A. RPV water level cannot be restored and maintained above +15 inches OR cannot be determined										Not Applicable										c. A. SAG entry is required.									
	Not Applicable										Not Applicable										Not Applicable										Not Applicable									
3 RCS Leak Rate	Not Applicable										Not Applicable										Not Applicable										Not Applicable									
	Not Applicable										Not Applicable										Not Applicable										Not Applicable									
4 Primary Containment Radiation	c. A. Drywell Monitor (9184A/B) reading greater than 200 R/hr OR c. B. Torus Monitor (9185A/B) reading greater than 200 R/hr										c. A. UNSOLUBLE break in Main Steam, HPCL, Feedwater, RWCU, or RCIC as indicated by the failure of both isolation valves in ANY one line to close AND EITHER: • High MSL flow or steam tunnel temperature annunciators OR • Direct report of steam release OR c. B. Emergency RPV Depressurization required.										c. A. Drywell Monitor (9184A/B) reading greater than 5 R/hr after reactor shutdown										Not Applicable									
	Not Applicable										Not Applicable										Not Applicable										Not Applicable									
5 Other Indications	c. A. Fuel damage assessment indicates at least 5% fuel clad damage.										Not Applicable										Not Applicable										Not Applicable									
	Not Applicable										Not Applicable										Not Applicable										Not Applicable									
6 Emergency Director Judgment	c. ANY condition in the opinion of the Emergency Director that indicates Loss of the Fuel Clad barrier.										c. ANY condition in the opinion of the Emergency Director that indicates Loss of the Fuel Clad barrier.										c. ANY condition in the opinion of the Emergency Director that indicates Loss of the RCS barrier.										c. ANY condition in the opinion of the Emergency Director that indicates Loss of the Containment barrier.									
	Not Applicable										Not Applicable										Not Applicable										Not Applicable									