

L-98-16  
Attachment 3

ATTACHMENT 3

PROPOSED LICENSE AMENDMENTS FOR  
ADMINISTRATIVE CHANGES TO FACILITY OPERATING LICENSES  
AND TECHNICAL SPECIFICATIONS

PROPOSED FACILITY OPERATING LICENSE PAGES DPR-31

pages 4 through 8

PROPOSED FACILITY OPERATING LICENSE PAGES DPR-41

pages 5 through 9

PROPOSED TECHNICAL SPECIFICATIONS PAGES

FIGURE 5.1-2

Section 6 page 6-6

Section 6 page 6-11

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H. Safeguards Contingency Plan

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I. Steam Generator Repair Program

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- (1) The Turkey Point Plant steam generator repair program, as described in the licensee's "Steam Generator Repair Report" dated September 20, 1977, as supplemented on December 20, 1977, March 7, April 25, June 20 and August 4, 1978, January 26, 1979 and March 28, 1980, and the affidavit of A. J. Gould dated June 12, 1981, for Unit No. 3 is approved pursuant to the Atomic Safety and Licensing Board Final Order dated June 19, 1981.
  - (2) During the repair program the following temporary license conditions\* will be imposed:
    - (a) All fuel shall be removed from the reactor pressure vessel of the unit under repair and stored in the spent fuel pool.
    - (b) The health physics program and procedures which have been established for the steam generator repair program shall be implemented.
    - (c) Progress reports shall be provided at 60-day intervals from the start of the repair program and due 30 days after close of the interval with a final report provided within 60 days after completion of the repair. These reports will include:
      - (i) A summary of the occupation exposure expended to date using the format and detail of Table 3.3-2 of the "Steam Generator Repair Report" as supplemented.
      - (ii) An evaluation of the effectiveness of dose reduction techniques as specified in Section 3.3.5 of the "Steam Generator Repair Report" as supplemented in reducing occupational exposures.
      - (iii) An estimate of radioactivity released in both liquid and gaseous effluents.
      - (iv) An estimate of the solid radioactive waste generated during the repair effort including volume and radioactive content.

\*Reference in parentheses refer to the Safety Evaluation Report (NUREG-0756) December, 1980.

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- (d) Procedures shall be prepared to assure that power can be restored by manual operator actions to the fuel pool of the unit undergoing repair within eight hours (3.2.2.2).
  - (e) The remedy chosen by FPL to provide the availability of the diesel fuel supply while the oil-retention dike is removed from the main diesel safety tank shall be addressed and adequately demonstrated by FPL prior to initiating the construction changes affecting the dike (3.2.2.2).
  - (f) Sixty days prior to fuel loading, the program for preoperational testing and startup shall be submitted for NRC review (2.7).
  - (g) Sixty days prior to fuel loading, FPL should submit for evaluation by the NRC a steam generator secondary water chemistry control and monitoring program (3.2.4) which will address the following:
    - (i) Identification of a sampling schedule for the critical parameters and of control points for these parameters for each mode of operation: normal operation, hot startup, cold startup, hot shutdown, cold wet layup;
    - (ii) Identification of the procedures used to measure the values of the critical parameters;
    - (iii) Identification of process sampling points;
    - (iv) Procedure for the recording and management of data;
    - (v) Procedures defining corrective actions\* for off-control point chemistry conditions; and
    - (vi) A procedure identifying (a) the authority responsible for the interpretation of the data and (b) the sequence and timing of administrative events required to initiate corrective action.

\* Branch Technical Position MTEB 5-3 describes the acceptable means for monitoring secondary side water chemistry in PWR steam generators, including corrective actions for off-control point chemistry conditions. However, the staff is amenable to alternatives, particularly to Branch Technical Position B.3.b(9) of MTEB 5-3 (96 hour time limit to repair or plug confirmed condenser tube leaks).

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FPL should verify that the steam generator secondary water chemistry control program incorporates technical recommendations of the NSSS vendor. Any significant deviations from NSSS vendor recommendations should be noted and justified technically.

(h) Sixty days prior to the decontamination of the channel head, FPL should meet the following conditions (3.2.5):

- (i) A system should be set up so that the pressure in the inflatable plug seal in the RCS pipe nozzles should be monitored. Upon loss of seal pressure, injection of the grit slurry should be stopped immediately and the seal plug replaced.
- (ii) Written procedures should be provided to include accountability controls of all tools, equipment, materials, and supplies that are to be used in the channel heads to prevent inadvertent entry of such items into the reactor primary coolant system. These controls should be in effect whenever the inflatable plug seals and their associated cover plates are not in place in the nozzles of the reactor coolant system piping.
- (iii) Written procedures should be provided to restrict materials to be used in the channel head area to prevent the presence of materials having potential adverse effects on the reactor coolant system components (for example, chloride-bearing materials).
- (iv) Written procedures should be provided to include instructions to require that the channel head area, including the nozzles, be inspected and confirmed to be free of all loose materials, equipment, and tools prior to removing the cover plate from the inflatable plug seal.
- (v) Prior to closing up the reactor coolant system and starting the RCS pumps, any loose debris, including the abrasive grits, in the channel head, RCS hot leg, and cold leg should be cleaned up.
- (vi) Prior to resumption of power operation, the licensee should submit for NRC review and acceptance a report which will include an analysis of the possible effects of any foreign material which has entered the primary coolant system and has not been retrieved. The report should include all work on the decontamination and steam generator repair.

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- (i) Sixty days prior to the movement of the used steam generator lower assemblies from the containment, the procedures for the move, associated QA requirements, and a description of the equipment to be used shall be provided to the NRC (3.2.6.).
  - (j) Before storage or shipment of the used steam generator lower assemblies, the seal welds must be coated with a heavy bodied varnish such as glyptol (3.2.6).
  - (k) If credit for the unplugged configuration of the repaired steam generators is to be taken, a new ECCS analysis using the approved model will be required (3.3.1).

J. Guard Training and Qualification Plan

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K. Integrated Schedule

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- 1. The Plan for Integrated Scheduling of Plant Modifications for Turkey Point Units 3 & 4 (the Plan), submitted on December 19, 1986, is approved.
    - a. The Plan shall be followed by the licensee from and after the effective date of this amendment.
    - b. Changes to dates for completion of items identified in Schedule B do not require a licensee amendment. Dates specified in Schedule A shall be changed only in accordance with applicable NRC procedures.
  - 2. This license condition shall be effective until December 31, 1991, subject to renewal upon application by the licensee.

- L. The license shall fully implement and maintain in effect all provisions of the Commission-approved physical security, guard training and qualification, and safeguards contingency plans including amendments made pursuant to provision of the Miscellaneous Amendments and Search Requirements revisions to 10 CFR 73.55 (51 FR 27817 and 27822) and to the authority of 10 CFR 50.90 and 10 CFR 50.54(p). The plans, which contain Safeguards Information protected under 10 CFR 73.21, are entitled: "Turkey Point Plant, Units 3 and 4 Security Plan," with revisions submitted through April 13, 1988; "Turkey Point Plant, Units 3 and 4, Training and Qualification Plan," with revisions submitted through December 18, 1986; and "Turkey Point Plant, Units 3 and 4 Safeguards Contingency Plan," with revisions submitted through July 15, 1985. Changes made in

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accordance with 10 CFR 73.55 shall be implemented in accordance with the schedule set forth therein.

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4. ~~7/97~~ FPL shall proceed with implementation of the recommendations set forth in paragraphs 7b and c of the "Summary and Conclusions" section of the "Final Environmental Statement Related to Operation of Turkey Point, Florida Power and Light Company, Docket Nos. 50-250 and 50-251," issued July 1972 by the AEC Directorate of Licensing. No later than thirty (30) days from the date of issuance of this license, FPL shall submit to the AEC, for review and approval, its plan for the implementation of such recommendations.

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8. This license is effective as of the date of issuance, and shall expire at midnight July 19, 2012.

F. Fire Protection

FPL shall implement and maintain in effect all provisions of the approved Fire Protection Program--as described in the Updated Final Safety Analysis Report (UFSAR) for Turkey Point Units 3 and 4 and as approved in the Safety Evaluation Report (SER) dated March 21, 1979 and supplemented by NRC letters dated April 3, 1980, July 9, 1980, December 8, 1980, January 26, 1981, May 10, 1982, March 27, 1984, April 16, 1984, August 12, 1987, and by Safety Evaluation dated February 25, 1994, subject to the following provision.

The licensee may make changes to the approved Fire Protection Program without prior approval of the Commission only if those changes would not adversely affect the ability to achieve and maintain safe shutdown in the event of a fire.

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- (2) During the repair program the following temporary license conditions\* will be imposed:
- (a) All fuel shall be removed from the reactor pressure vessel of the unit under repair and stored in the spent fuel pool.
  - (b) The health physics program and procedures which have been established for the steam generator repair program shall be implemented.
  - (c) Progress reports shall be provided at 60-day intervals from the start of the repair program and due 30 days after close of the interval with a

\*References in parentheses refer to the Safety Evaluation Report (NUREG-0756) December 1980.





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final report provided within 60 days after completion of the repair. These reports will include:

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- (i) A summary of the occupation exposure expended to date using the format and detail of Table 3.3-2 of the "Steam Generator Repair Report" as supplemented.
  - (ii) An evaluation of the effectiveness of dose reduction techniques as specified in Section 3.3.5 of the "Steam Generator Repair Report" as supplemented in reducing occupational exposures.
  - (iii) An estimate of radioactivity released in both liquid and gaseous effluents.
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    - (I) Identification of a sampling schedule for the critical parameters and of control points for these parameters for each mode of operation: normal operation, hot startup, cold startup, hot shutdown, cold wet layup;
    - (ii) Identification of the procedures used to measure the values of the critical parameters;

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- (iii) Identification of process sampling points;
- (iv) Procedure for the recording and management of data;
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- (vi) A procedure identifying (a) the authority responsible for the interpretation of the data and (b) the sequence and timing of administrative events required to initiate corrective action.

FPL should verify that the steam generator secondary water chemistry control program incorporates technical recommendations of the NSSS vendor. Any significant deviations from NSSS vendor recommendations should be noted and justified technically.

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- (i) A system should be set up so that the pressure in the inflatable plug seal in the RCS pipe nozzles should be monitored. Upon loss of seal pressure, injection of the grit slurry should be stopped immediately and the seal plug replaced.
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- (i) Sixty days prior to the movement of the used steam generator lower assemblies from the containment, the procedures for the move, associated QA requirements, and a description of the equipment to be used shall be provided to the NRC (3.2.6).
- (j) Before storage or shipment of the used steam generator lower assemblies, the seal welds must be coated with a heavy bodied varnish such as glyptol (3.2.6).
- (k) If credit for the unplugged configuration of the repaired steam generators is to be taken, a new ECCS analysis using the approved model will be required (3.3.1).

I. Guard Training and Qualification Plan

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J. IAEA Safeguards

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K. Integrated Schedule ~~DELETED~~

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  2. This license condition shall be effective until December 31, 1991, subject to renewal upon application by the licensee.

L. The license shall fully implement and maintain in effect all provisions of the Commission-approved physical security, guard training and qualification, and safeguards contingency plans including amendments made pursuant to provision of the Miscellaneous Amendments and Search Requirements revisions to 10 CFR 73.55 (51 FR 27817 and 27822) and to the authority of 10 CFR 50.90 and 10 CFR 50.54(p). The plans, which contain Safeguards Information protected under 10 CFR 73.21, are entitled: "Turkey Point Plant, Units 3 and 4 Security Plan," with revisions submitted through April 13, 1988; "Turkey Point Plant, Units 3 and 4, Training and Qualification Plan," with revisions submitted through December 18, 1986; and "Turkey Point Plant, Units 3 and 4 Safeguards Contingency Plan," with revisions submitted through July 15, 1985. Changes made in accordance with 10 CFR 73.55 shall be implemented in accordance with the schedule set forth therein.

4. This license is effective as of the date of issuance, and shall expire at midnight April 10, 2013.



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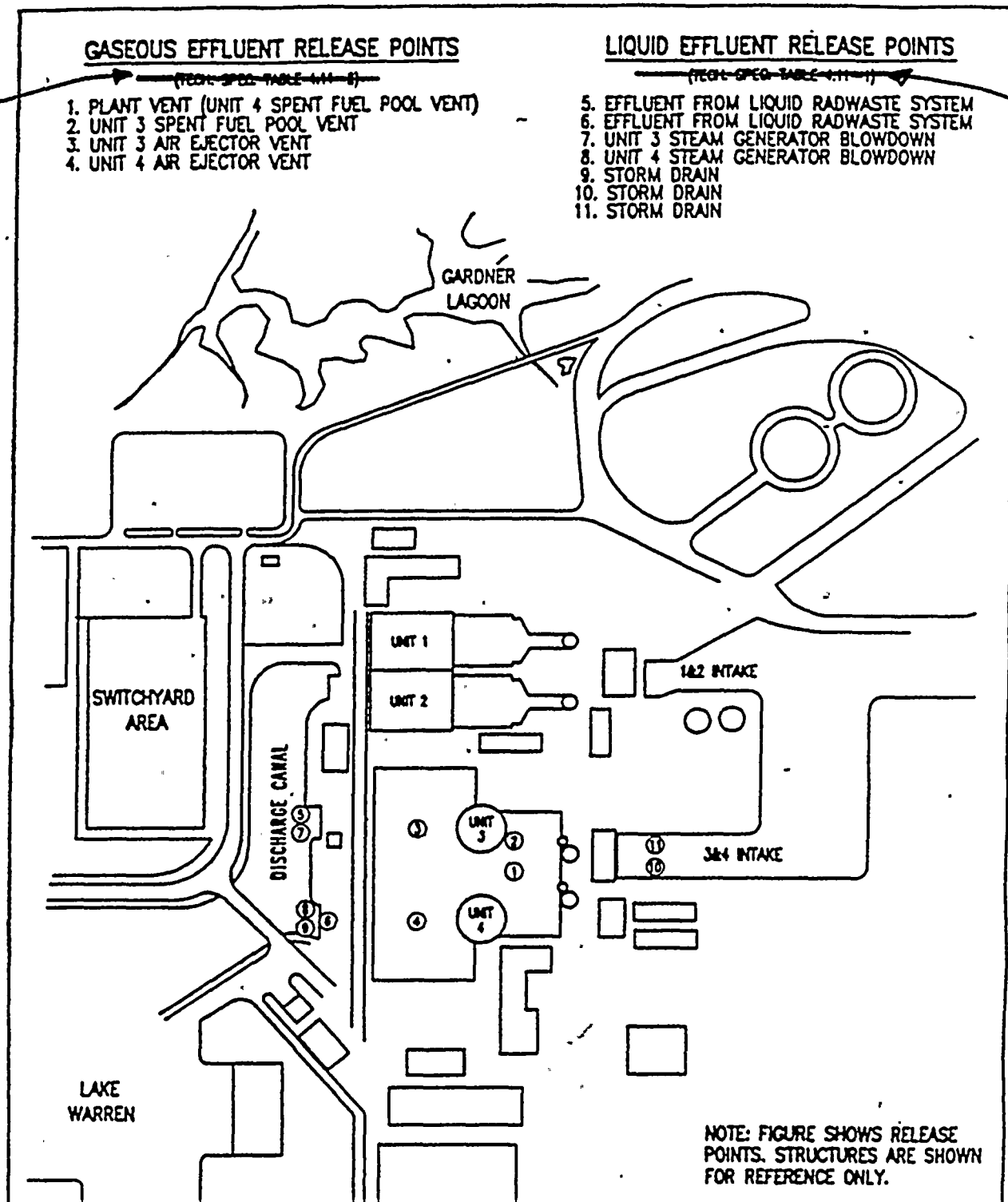


FIGURE 5.1-2 PLANT AREA MAP

## ADMINISTRATIVE CONTROLS

### COMPOSITION

6.5.1.2 The PNSC shall have a minimum of nine voting members and be composed of individuals from each of the following disciplines:

Operations  
Maintenance  
Health Physics  
Reactor Engineering  
Services

Technical Support  
Licensing  
Quality Assurance/Control  
Instrument and Control

Add

Protection

The PNSC Chairman and Vice-Chairman shall be appointed in writing from among the members by the Plant General Manager.

The members, according to individual job titles, shall meet the requirements as described in Sections 4.2, 4.3, or 4.4, of the ANSI N-18.1-1971.

### ALTERNATES

6.5.1.3 All alternate members shall be appointed in writing by the Plant General Manager to serve on a temporary basis; however, no more than two alternates shall participate as members in PNSC activities at any one time.

### MEETING FREQUENCY

6.5.1.4 The PNSC shall meet at least once per calendar month and as convened by the PNSC Chairman or his designated alternate.

### QUORUM

6.5.1.5 The quorum of the PNSC necessary for the performance of the PNSC responsibility and authority provisions of these Technical Specifications shall consist of the Chairman or Vice Chairman and four members including alternates.

### RESPONSIBILITIES

6.5.1.6 The PNSC shall be responsible for:

- a. Review of all safety-related plant administrative procedures and changes thereto.
- b. Review of all proposed tests and experiments that affect nuclear safety;
- c. Review of all proposed changes to Appendix "A" Technical Specifications;
- d. Review of all proposed changes or modifications to unit systems or equipment that affect nuclear safety;



## ADMINISTRATIVE CONTROLS

### RECORDS

6.5.2.9 Records of CNRB activities shall be prepared, approved, and distributed as indicated below:

- a. Minutes of each CNRB meeting shall be prepared, approved, and forwarded to the President-Nuclear Division within 14 days following each meeting;
- b. Reports of reviews encompassed by Specification 6.5.2.7 shall be prepared, approved, and forwarded to the President-Nuclear Division within 14 days following completion of the review; and
- c. Audit reports encompassed by Specification 6.5.2.8 shall be forwarded to the President-Nuclear Division and to the management positions responsible for the areas audited within 30 days after completion of the audit by the auditing organization.

### 6.5.3 TECHNICAL REVIEW AND CONTROL

#### ACTIVITIES

6.5.3.1 Activities that affect nuclear safety shall be conducted as follows:

- a. Procedures required by Specification 6.8, and other procedures that affect nuclear safety, and changes thereto, shall be prepared, reviewed, and approved. Each such procedure, or change thereto, shall be reviewed by an individual/group other than the individual/group who prepared the procedure, or change thereto, but who may be from the same organization as the individual/group who prepared the procedure, or change thereto. Procedures other than plant administrative procedures shall be approved by the Plant General Manager, Operations Manager, or the head of the department assigned responsibility for those procedures prior to implementation. The Plant General Manager shall approve plant administrative procedures and emergency plan implementing procedures. Security Plan and the implementing procedures shall be approved by Services Manager prior to implementation. Changes to procedures that may involve a change to the intent of the original procedures shall be approved by the individual authorized to approve the procedure prior to implementation of the change.

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Protection

