CATEGORY 1

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AUTH.NAME AUTHOR AFFILIATION
HOVEY,R.J. Florida Power & Light Co.
RECIP.NAME RECIPIENT AFFILIATION
PEEBLES,T.A. Region 2 (Post 820201)

SUBJECT: Notifies of FPL shipment of operator license exams matl requested by NRC 970523 ltr to listed examiner.Index of reference matl,encl.

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CATEGORY 1 DOCUMENT

FPL

JUL 29 1997

L-97-192

Mr. Thomas A. Peebles, Chief
Operator Licensing and Human Performance Branch
Division of Reactor Safety, Region II
U. S. Nuclear Regulatory Commission
Atlanta Federal Center
61 Forsyth Street, S. W., Suite 23T85
Atlanta, GA 30303

Dear Mr. Peebles:

Re: Turkey Point Units 3 and 4
Docket Nos. 50-250 and 50-251
Operator License Examinations
Scheduled for the Week of September 5, 1997

This letter is notification of the Florida Power & Light Company (FPL) shipment of reference material requested by NRC letter dated May 23, 1997, to the examiner and address listed below:

Attn: Mr. C. Payne
U. S. Nuclear Regulatory, Commission, Region II
Atlanta Federal Center
61 Forsyth Street, S. W., Suite 23T85
Atlanta, GA 30303

The reference material was shipped to NRC Region II, on July 29, 1997. An index of the reference material is attached for your information.

The examiners are requested to retain the reference material so it can be used during the license examination scheduled for September 1997. Should you require any additional information, please contact Mr. Bill Miller at (305) 246-6650.

Very truly yours,

DE Jernigan for

Robert J. Hovey
Vice President
Turkey Point Plant

OIH

Attachment

cc: L. A. Reyes, Regional Administrator, Region II, USNRC
Document Control Desk, USNRC
T. P. Johnson, Senior Resident Inspector, USNRC, Turkey Point Plant

IE42

PTN REFERENCE MATERIALS - SRO UPGRADE '97

PLANT PROCEDURES

Off-Normal Operating Procedures (2 Books)
Emergency Operating Procedures (2 Books)
EOP Basis Documents (3 Books)
Emergency Plan Implementing Procedures

OTHER PLANT REFERENCE/TRAINING MATERIALS

Technical Specifications with Position Statements and Basis
E-Plan

OFF-NORMAL OPERATING PROCEDURES

Proced.#	Title
3-ONOP-003.4	Loss of DC Bus 3D01 and 3D01A (3A)
3-ONOP-003.5	Loss of DC Bus 3D23 and 3D23A (3B)
3-ONOP-003.6	Loss of 120 V Vital Inst Panel 3P06
3-ONOP-003.7	Loss of 120 V Vital Inst Panel 3P07
3-ONOP-003.8	Loss of 120 V Vital Inst Panel 3P08
3-ONOP-003.9	Loss of 120 V Vital Inst Panel 3P09
3-ONOP-004	Loss of Offsite Power
3-ONOP-004.1	System Restoration Following Loss of Offsite Power
3-ONOP-004.2	Loss of 3A 4KV Bus
3-ONOP-004.3	Loss of 3B 4KV Bus
3-ONOP-004.4	Loss of 3C 4KV Bus
3-ONOP-004.5	Loss of 3D 4KV Bus
3-ONOP-008	Turbine Plant Cooling Water Malfunction
3-ONOP-011	Screen Wash System / Intake Malfunction
0-ONOP-013	Loss of Instrument Air
3-ONOP-014	Main Condenser Loss of Vacuum
0-ONOP-016.7	Screen Wash Emergency Makeup to the Fire Protection System
3-ONOP-019	Intake Cooling Water Malfunction
3-ONOP-023.2	Emergency Diesel Generator Failure
3-ONOP-028	Reactor Control System Malfunction
3-ONOP-028.1	RCC Misalignment
3-ONOP-028.2	RCC Position Indication Malfunction
3-ONOP-028.3	Dropped RCC
3-ONOP-030	Component Cooling Water Malfunction
3-ONOP-033.1	Spent Fuel Pit (SFP) Cooling System Malfunction
3-ONOP-033.2	Refueling Cavity Seal Failure
3-ONOP-033.3	Accidents Involving New or Spent Fuel
3-ONOP-041.1	Reactor Coolant Pump Off-Normal
3-ONOP-041.3	Excessive Reactor Coolant System Leakage
3-ONOP-041.4	Excessive Reactor Coolant System Activity
3-ONOP-041.5	Pressurizer Pressure Control Malfunction
3-ONOP-041.6	Pressurizer Level Control Malfunction
3-ONOP-041.7	Shutdown LOCA [Mode 3 (less than 1000psig) or Mode 4]
3-ONOP-041.8	Shutdown LOCA [Mode 5 or 6]
3-ONOP-046.1	Emergency Boration
0-ONOP-046.3	Loss of Boration Flowpath(s)
3-ONOP-046.4	Malfunction of Boron Concentration Control System
3-ONOP-047.1	Loss of Charging Flow in Modes 1 Through 4
3-ONOP-049.1	Deviation or Failure of Safety Related or Reactor Protection Channels
3-ONOP-050	Loss of RHR
3-ONOP-053	Loss of Containment Integrity

OFF-NORMAL OPERATING PROCEDURES (continued)

Proced.#	Title
3-ONOP-059.4	Excessive Axial Flux Difference
3-ONOP-059.5	Source Range Nuclear Instrumentation Malfunction
3-ONOP-059.6	Backup NIS (Gamma Metrics) Malfunction
3-ONOP-059.7	Intermediate Range Nuclear Instrumentation Malfunction
3-ONOP-059.8	Power Range Nuclear Instrumentation Malfunction
3-ONOP-059.9	Excessive Quadrant Power Tilt Ratio
0-ONOP-066	High Area Radiation Monitoring System Alarm
3-ONOP-067	Radioactive Effluent Release
3-ONOP-071.1	Secondary Chemistry Deviation from Limits
3-ONOP-075	Auxiliary Feedwater System Malfunction
3-ONOP-089	Turbine Runback
3-ONOP-092.1	Main Transformer Malfunction
3-ONOP-092.2	Auxiliary Transformer Malfunction
3-ONOP-092.3	Startup Transformer Malfunction
3-ONOP-092.4	C Bus Transformer Malfunction
3-ONOP-100	Fast Load Reduction
0-ONOP-105	Control Room Evacuation

UNIT 3 EMERGENCY OPERATING PROCEDURES

Proced.#	Title
3-EOP-E-0	Reactor Trip or Safety Injection
3-EOP-ES-0.0	Rediagnosis
3-EOP-ES-0.1	Reactor Trip Response
3-EOP-ES-0.2	Natural Circulation Cooldown
3-EOP-ES-0.3	Natural Circulation Cooldown With Steam Void In Vessel (With RVLMS)
3-EOP-ES-0.4	Natural Circulation Cooldown With Steam Void In Vessel (W/O RVLMS)
3-EOP-E-1	Loss of Reactor or Secondary Coolant
3-EOP-ES-1.1	SI Termination
3-EOP-ES-1.2	Post LOCA Cooldown and Depressurization
3-EOP-ES-1.3	Transfer to Cold Leg Recirculation
3-EOP-ES-1.4	Transfer to Hot Leg Recirculation
3-EOP-E-2	Faulted Steam Generator Isolation
3-EOP-E-3	Steam Generator Tube Rupture
3-EOP-ES-3.1	Post-SGTR Cooldown Using Backfill
3-EOP-ES-3.2	Post-SGTR Cooldown Using Blowdown
3-EOP-ES-3.3	Post-SGTR Cooldown Using Steam Dump
3-EOP-ECA-0.0	Loss of All AC Power
3-EOP-ECA-0.1	Loss of All AC Power Recovery Without SI Required
3-EOP-ECA-0.2	Loss of All AC Power Recovery With SI Required
3-EOP-ECA-1.1	Loss of Emergency Coolant Recirculation
3-EOP-ECA-1.2	LOCA Outside Containment
3-EOP-ECA-2.1	Uncontrolled Depressurization of All Steam Generators
3-EOP-ECA-3.1	SGTR With Loss of Reactor Coolant - Subcooled Recovery Required
3-EOP-ECA-3.2	SGTR With Loss of Reactor Coolant - Saturated Recovery Desired
3-EOP-ECA-3.3	SGTR Without Pressurizer Pressure Control
3-EOP-F-0	Critical Safety Function Status Trees
3-EOP-FR-S.1	Response to Nuclear Power Generation/ATWS
3-EOP-FR-S.2	Response to Loss of Core Shutdown
3-EOP-FR-C.1	Response to Inadequate Core Cooling
3-EOP-FR-C.2	Response to Degraded Core Cooling
3-EOP-FR-C.3	Response to Saturated Core Cooling
3-EOP-FR-H.1	Response to Loss of Secondary Heat Sink
3-EOP-FR-H.2	Response to Steam Generator Overpressure
3-EOP-FR-H.3	Response to Steam Generator High Level
3-EOP-FR-H.4	Response to Loss of Normal Steam Release Capabilities
3-EOP-FR-H.5	Response to Steam Generator Low Level
3-EOP-FR-P.1	Response to Imminent Pressurized Thermal Shock Condition
3-EOP-FR-P.2	Response to Anticipated Pressurized Thermal Shock Condition
3-EOP-FR-Z.1	Response to High Containment Pressure
3-EOP-FR-Z.2	Response to Containment Flooding
3-EOP-FR-Z.3	Response to High Containment Radiation Level
3-EOP-FR-I.1	Response to High Pressurizer Level
3-EOP-FR-I.2	Response to Low Pressurizer Level
3-EOP-FR-I.3	Response to Voids in Reactor Vessel

UNIT 3 EMERGENCY OPERATING PROCEDURES - *BASIS DOCUMENTS*

Proced.#	Title
3-BD-EOP-E-0	Reactor Trip or Safety Injection
3-BD-EOP-ES-0.0	Rediagnosis
3-BD-EOP-ES-0.1	Reactor Trip Response
3-BD-EOP-ES-0.2	Natural Circulation Cooldown
3-BD-EOP-ES-0.3	Natural Circulation Cooldown With Steam Void in Vessel With RVLMS (QSPDS)
3-BD-EOP-ES-0.4	Natural Circulation Cooldown With Steam Void in Vessel (Without RVLMS)
3-BD-EOP-E-1	Loss of Reactor or Secondary Coolant
3-BD-EOP-ES-1.1	SI Termination
3-BD-EOP-ES-1.2	Post LOCA Cooldown and Depressurization
3-BD-EOP-ES-1.3	Transfer to Cold Leg Recirculation
3-BD-EOP-ES-1.4	Transfer to Hot Leg Recirculation
3-BD-EOP-E-2	Faulted Steam Generator Isolation
3-BD-EOP-E-3	Steam Generator Tube Rupture
3-BD-EOP-ES-3.1	Post-SGTR Cooldown Using Backfill
3-BD-EOP-ES-3.2	Post-SGTR Cooldown Using Blowdown
3-BD-EOP-ES-3.3	Post-SGTR Cooldown Using Steam Dump
3-BD-EOP-ECA-0.0	Loss of All AC Power
3-BD-EOP-ECA-0.1	Loss of All AC Power Recovery Without SI Required
3-BD-EOP-ECA-0.2	Loss of All AC Power Recovery With SI Required
3-BD-EOP-ECA-1.1	Loss of Emergency Coolant Recirculation
3-BD-EOP-ECA-1.2	LOCA Outside Containment
3-BD-EOP-ECA-2.1	Uncontrolled Depressurization of All Steam Generators
3-BD-EOP-ECA-3.1	SGTR With Loss of Reactor Coolant - Subcooled Recovery Desired
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3-BD-EOP-FR-P.2	Response to Anticipated Pressurized Thermal Shock Condition
3-BD-EOP-FR-Z.1	Response to High Containment Pressure
3-BD-EOP-FR-Z.2	Response to Containment Flooding
3-BD-EOP-FR-Z.3	Response to High Containment Radiation Level
3-BD-EOP-FR-I.1	Response to High Pressurizer Level
3-BD-EOP-FR-I.2	Response to Low Pressurizer Level
3-BD-EOP-FR-I.3	Response to Voids in Reactor Vessel

EMERGENCY PLAN IMPLEMENTING PROCEDURES

Proced.#	Title
EPIP-20101	Duties of Emergency Coordinator
EPIP-20104	Emergency Response Organization Notification/Staff Augmentation
EPIP-20106	Natural Emergencies
EPIP-20107	Fire/Explosion Emergencies
EPIP-20110	Criteria For, and Conduct of Owner Controlled Area Evacuation
EPIP-20111	Re-entry
EPIP-20126	Off-Site Dose Calculations