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 FACIL: 50-250 Turkey Point Plant, Unit 3, Florida Power and Light C 05000250
 50-251 Turkey Point Plant, Unit 4, Florida Power and Light C 05000251

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SUBJECT: Discusses change to Westinghouse NOTRUMP small break LOCA
 evaluation model per requirements of 10CFR50.46(a)(3)(ii).

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JAN 26 1994

L-94-015
10 CFR 50.46

U. S. Nuclear Regulatory Commission
Attn: Document Control Desk
Washington, D.C. 20555

Re: Turkey Point Units 3 and 4
Docket Nos. 50-250 and 50-251
10 CFR 50.46, "Acceptance Criteria for
Emergency Core Cooling Systems in
Light Water Power Reactors"

Gentlemen:

By letter L-93-267, dated October 29, 1993, Florida Power and Light (FPL) Company submitted the annual 10 CFR 50.46 report which summarized the limiting emergency core cooling system (ECCS) analyses for Turkey Point Units 3 and 4. In letter L-93-267, FPL reported a change to the Westinghouse NOTRUMP small break Loss of Coolant Accident (LOCA) evaluation model, which is currently used in the licensing analysis for Turkey Point. Since the model change has been determined to be significant, a plan for reanalysis, as required by 10 CFR 50.46(a)(3)(ii), is discussed below.

During a meeting between Westinghouse and the NRC on January 12, 1994, Westinghouse agreed to submit to the NRC for review and approval an addendum to WCAP-10054-P-A, "Westinghouse Small Break ECCS Evaluation Model using the NOTRUMP Code", describing the revised safety injection (SI) model used in NOTRUMP, including SI to the broken loop. The addendum will reference the improved condensation model (COSI) described in Westinghouse Topical Report WCAP-11767 and provide justification for application of the COSI model to small break LOCA calculations. Since the NRC review may affect the changes to the NOTRUMP evaluation model, FPL does not plan to reanalyze the SI in the broken loop event prior to completion of the NRC review. In the interim, FPL will consider the Peak Clad Temperature (PCT) change reported to the NRC in FPL letter L-93-267 as a permanent change to Turkey Point's calculated PCT. Once the NRC has approved the NOTRUMP addendum on SI in the broken loop, and following Westinghouse's incorporation into the NOTRUMP coding, FPL plans to capture the new model at the next licensing action requiring re-analysis of the small break LOCA. If Westinghouse elects not to change the coding of the NOTRUMP evaluation model, then FPL plans to not reanalyze but to report the net effect on PCT identified in the NRC's Safety Evaluation Report (SER) for the addendum to WCAP-10054-P-A, as a permanent change at the next required 10 CFR 50.46 report.

Should there be any questions, please contact us.

Very truly yours,

T. F. Plunkett
Vice President
Turkey Point Plant

TEP/RJT/rt

cc: S. D. Ebnetter, Regional Administrator, Region II, USNRC
T. P. Johnson, Sr. Resident Inspector, USNRC, Turkey Point Plant

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