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~~Jan-June 1992~~ " W/920829 ltr.

July 1, 1991 Thru June 30, 1992

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10 CFR 50.59

U. S. Nuclear Regulatory Commission
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Gentlemen:

Re: Turkey Point Units 3 and 4
Docket No. 50-250 and 50-251
10 CFR 50.59 Report

Florida Power & Light Company's Report on "Changes, Tests and Experiments Made Without Prior Commission Approval" for the period July 1, 1991 through June 30, 1992 is attached.

Very truly yours,

T. F. Plunkett
Vice President
Turkey Point Nuclear

TFP/OIH/oh

Attachment

cc: Stewart D. Ebnetter, Regional Administrator, Region II, USNRC
R. C. Butcher, Senior Resident Inspector, USNRC, Turkey Point Plant

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***ANNUAL
10 CFR 50.59
REPORT***

***FLORIDA POWER & LIGHT COMPANY
TURKEY POINT UNITS 3 & 4***

TURKEY POINT PLANT UNITS 3 AND 4
DOCKET NUMBERS 50-250 AND 50-251
CHANGES, TESTS AND EXPERIMENTS
MADE AS ALLOWED BY 10 CFR 50.59
FOR THE PERIOD OF
JULY 1, 1991 THROUGH JUNE 30, 1992



INTRODUCTION

This report is submitted in accordance with 10 CFR 50.59(b), which requires that:

- i) changes in the facility as described in the SAR
- ii) changes in procedures as described in the SAR, and
- iii) tests and experiments not described in the SAR

which are conducted without prior Commission approval be reported to the Commission at least annually. This report is intended to meet this requirement for the period of July 1, 1991, through June 30, 1992.

This report is divided into five (5) sections; the first, changes to the facility as described in the SAR performed by a Plant Change/Modification (PC/M); the second, changes to the facility or procedures as described in the SAR not performed by a PC/M and tests and experiments not described in the SAR; the third, a summary of any fuel reload evaluations; the fourth, a list of Power Operated Relief Valve (PORV) actuations, which is submitted in accordance with FPL's commitment to comply with the requirements of Item II.K.3.3 of NUREG 0737; the fifth, a summary of the findings of Steam Generator tube inspections. Unit 3 and Unit 4 did not have a Steam Generator tube inspection during this reporting period.



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PLANT CHANGE/MODIFICATION 76-01

UNIT : 3/4
TURN OVER DATE : 03/12/92

HYDRAZINE PUMP CONTROL

Summary:

This Engineering Package provides for the installation of a Hydrazine Analyzer in the steam generator feedwater sample line to control hydrazine pump speed in order to automatically maintain the hydrazine level at a desired setpoint of 25 ppb.

At present the hydrazine pump speed is proportional to feedwater flow. At a fixed power level, flow is constant regardless of dissolved oxygen in the feedwater. If a feedwater leak develops, the hydrazine pump speed remains constant and oxygen concentration increases, tending to increased corrosion, etc. With this new method of continuous hydrazine monitoring, the pump speed will immediately go up to maintain a desired hydrazine concentration. This will limit the buildup of dissolved oxygen.

Safety Evaluation:

This modification did not have any adverse effect on the plant safety or operation. This modification did not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.



PLANT CHANGE/MODIFICATION 81-34-03

UNIT : 3/4
TURN OVER DATE : 05/07/92

INSTALLATION OF THE NUCLEAR DATA ACCOUNTING SYSTEM

Summary:

This Engineering Package provided for the installation of the Nuclear Data Accounting System (NDAS) within the computer room and the interconnecting cables for the interface between the NDAS and the Automatic Isotope Measurement System (AIMS) which is part of the post accident sampling system and is located in the Radiochemistry Laboratory. Also required are the connections between the NDAS and the CRT, line printer and the existing Germanium Detectors, all located in the Radiochemistry Laboratory. The NDAS is required to provide meteorological and radiological release reports required by the NRC in accordance with 10 CFR 50, Appendix I.

Safety Evaluation:

The Nuclear Data Accounting System has no interconnections with any safety related system. This system does not perform any nuclear safety related function and does not affect any equipment as described in the FSAR. This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.



PLANT CHANGE/MODIFICATION 84-165-07

UNIT : 3/4
TURN OVER DATE : 08/31/91

GAS DECAY TANK COMBUSTIBLE GAS MONITORING PANEL CHANGES

Summary:

The changes described in this PC/M provide the design documentation necessary to install a dedicated, single point, gas analyzer panel (C289) for continuous monitoring and recording of oxygen and hydrogen concentrations of gas sampled from the in-service Gas Decay Tank. The panel is required in order to meet Radiological Effluent Technical Specification (RETS) requirements per NUREG 0472.

Safety Evaluation:

Panel C289 is not part of the Reactor Coolant System pressure boundary and is not required to be functional for accident prevention/mitigation or safe shutdown. PC/M 84-165 may be implemented during any Plant mode and will have no effect on Plant safety, security or operations. This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.

PLANT CHANGE/MODIFICATION 85-085-02

UNIT : 3/4
TURN OVER DATE : 03/03/92

AUXILIARY FEEDWATER CONTROL VALVE INSTRUMENT AIR FILTRATION
MODIFICATION

Summary:

This Engineering Package provides for the installation of new filters in the instrument air (nitrogen backed) supply line to each Unit 3 Auxiliary Feedwater Control Valve. The installation of the filters provides better quality air to the Control Valve positioners and actuators. The proposed system will provide a better filtration mechanism which can be isolated from the system without affecting the operability of the system. Therefore, subsequent maintenance on the new filters can be done without recalibration or retesting of the valve control system.

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.

PLANT CHANGE/MODIFICATION 85-135-07

UNIT : 3
TURN OVER DATE : 09/10/91

MSIV AIR OPERATOR NITROGEN BACKUP SUPPLY MODIFICATION

Summary:

This Engineering Package provides for a dedicated safety related nitrogen supply subsystem for each MSIV, consisting of independent pneumatic circuits, redundant electric control solenoid valves and dedicated, high pressure gas reserves which ensures each MSIV will close in five seconds or less upon receipt of a closure signal coincident with the most limiting process conditions for valve closure. In addition, the modification ensures MSIV closure can be maintained for a minimum of one hour without the need for operator action, independent of the availability of instrument air.

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.

PLANT CHANGE/MODIFICATION 85-176-05

UNIT : 4
TURN OVER DATE : 11/04/91

AFW NITROGEN BACKUP SYSTEM STATION RELOCATION AND ADDITION

Summary:

This Engineering Package provides for the relocation and addition of redundant AFW Nitrogen Backup System Stations. The existing AFW Nitrogen Backup System will be upgraded and seismically installed. The nitrogen bottle stations will be located within security fencing. The existing instruments will be replaced with new seismically qualified instruments. New seismically qualified rupture disks provide over-pressure protection in the event of a pressure regulator failure. Personnel safety will be enhanced by reorienting the flexible hose end connections toward the rear of the bottle racks and orienting the hose U-bends away from the front of the rack.

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.



PLANT CHANGE/MODIFICATION 86-006-02

UNIT : 3
TURN OVER DATE : 09/04/91

REMOVAL OF UNIT 3 LOAD FREQUENCY CONTROL EQUIPMENT

Summary:

This Engineering Package provides the design for four plant modifications at Turkey Point Unit 3. These modifications include the removal of the load frequency control equipment, modification of underpower relaying in the load drop anticipator circuits, removal of a frequency recorder from the Control Room, and replacement of the obsolete watt and var transducers used for telemetering with combination watt/var transducers.

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.

PLANT CHANGE/MODIFICATION 86-007-01

UNIT : 4
TURN OVER DATE : 08/20/91

REMOVAL OF UNIT 4 LOAD FREQUENCY CONTROL EQUIPMENT

Summary:

This Engineering Package provides the design for three plant modifications at Turkey Point Unit 4. These modifications include the removal of the load frequency control equipment, modification of underpower relaying in the load drop anticipator circuits, and replacement of the obsolete watt and var transducers used for telemetering with combination watt/var transducers.

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.

PLANT CHANGE/MODIFICATION 86-155-01

UNIT : 3
TURN OVER DATE : 08/23/91

UNIT 3 ADDITION OF EDG AIR START MOTORS

Summary:

This Engineering Package provides for the addition of a second Air Start Motor System for each existing Emergency Diesel Generator (EDG). The EDGs and associated equipment are necessary to safely shutdown the Plant and maintain it in a safe shutdown condition in the event of a loss of offsite power. Each existing EDG has two air start motors. If one of these motors fails, the EDG will not start, therefore, installation of the additional air start motors will enhance EDG reliability and upgrade the EDG Air Start System.

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.



PLANT CHANGE/MODIFICATION 86-200-05

UNIT : 3
TURN OVER DATE : 01/21/92

UNIT 3 HIGH INITIAL RESPONSE EXCITATION SYSTEM

Summary:

This Engineering Package provides for the modifications to the Turbine-Generator brushless excitation system. The brushless excitation system will be upgraded to a High Initial Response (HIR) Brushless Excitation System which will allow the generator to respond quickly to changes in system voltage.

A larger permanent magnet generator, a new stator coil in the brushless exciter, and internal modifications to the existing voltage regulator will be required to modify this system. The exciter bearing pedestal (#9 bearing) will be replaced with a double insulated pedestal. Modifications to the Control Room are minor and include reengraving of annunciator windows and replacement of the exciter field voltmeter and ammeter.

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.

PLANT CHANGE/MODIFICATION 86-213-03

UNIT : 3/4
TURN OVER DATE : 06/09/92

EMERGENCY LIGHTING FOR APPENDIX R

Summary:

This Engineering Package will provide the necessary illumination to meet the requirements of 10CFR50, Appendix R, Section III.J and provide additional lighting facilities for operator convenience. This engineering package also provides layout drawings depicting Appendix R equipment and lights.

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.



PLANT CHANGE/MODIFICATION 86-237

UNIT : 3
TURN OVER DATE : 08/02/91

UNIT 3 SAFETY RELIEF VALVE REPLACEMENT - COMPONENT COOLING WATER
SYSTEM

Summary:

This Engineering Package covers the replacement of safety relief valves on the component cooling water system. These replacements are the result of normal maintenance. Replacement components are no longer available for the original valves. Further, these valves are being changed from threaded connections to flange connections to facilitate removal for maintenance and testing. The replacement valves have been manufactured in accordance with ASME Section VIII and have been determined to be equal to or better than the existing valves. The replacement valves can be considered a one-for-one replacement and will not adversely affect the plant safety or operation.

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.

PLANT CHANGE/MODIFICATION 87-008-03

UNIT : 3
TURN OVER DATE : 08/28/91

UNIT 3 NRC IE BULLETIN 83-03 MOV POSITION INDICATION

Summary:

This Engineering Package provides for modifications to safety related motor operated valves (MOV's) in the Auxiliary Feedwater (AFW) and the High Head Safety Injection (HHSI) systems and provides for selecting the thermal overload heater sizes to the applicable MOVs.

This Engineering Package will provide for the conversion of the MOV from two (2) rotors to four (4) rotors and the engineering and design details required to implement the close to open torque bypass switch and closed position indication wiring modifications of the applicable MOVs. This EP also provides the thermal overload heater selection for the motor operated valves which are required to operate under high differential pressure conditions.

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.

PLANT CHANGE/MODIFICATION 87-025

UNIT : 3
TURN OVER DATE : 08/27/91

UNIT 3 COMPONENT COOLING WATER SYSTEM NORMAL CONTAINMENT COOLER
TUBE BUNDLE REPLACEMENT

Summary:

This Engineering Package provides details for the replacement of the tube bundles in the Normal Containment Coolers (NCCs) in the Component Cooling Water (CCW) system. The new coolers have the capability to remove 2.0 million Btu per hour per cooler. These new coolers will enable the containment to maintain a lower overall temperature while the unit is operating at 100% power. This is especially critical for operation during the summer season.

This EP also provides for replacement of the sheaves on the NCC fan motors in order to increase the fan speed and maintain air flows, essentially, the same as before. Also, as a result of the increase in CCW flow through the new tube bundles, the CCW flow instrumentation has been upgraded for the higher flow rate. In addition, the resulting increase in horsepower of the NCC fans and the CCW pumps has been evaluated.

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.



PLANT CHANGE/MODIFICATION 87-026

UNIT : 4
TURN OVER DATE : 08/13/91

UNIT 4 REPLACEMENT OF NORMAL CONTAINMENT COOLER TUBE BUNDLES

Summary:

This Engineering Package provides details for the replacement of the tube bundles in the Normal Containment Coolers (NCCs) in the Component Cooling Water (CCW) system. The new coolers have the capability to remove 2.0 million Btu per hour per cooler. These new coolers will enable the containment to maintain a lower overall temperature while the unit is operating at 100% power. This is especially critical for operation during the summer season.

This EP also provides for replacement of the sheaves on the NCC fan motors in order to increase the fan speed and maintain air flows, essentially, the same as before. Also, as a result of the increase in CCW flow through the new tube bundles, the CCW flow instrumentation has been upgraded for the higher flow rate. In addition, the resulting increase in horsepower of the NCC fans and the CCW pumps has been evaluated.

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.



PLANT CHANGE/MODIFICATION 87-109

UNIT : 3/4
TURN OVER DATE : 08/20/91

DC EQUIPMENT/INVERTER ROOMS HVAC

Summary:

This Engineering Package provides for modification of the DC Equipment/Inverter Rooms HVAC system. The system will be upgraded to ensure that the safety related inverters do not experience temperatures above their qualifications during normal operation. The modifications will enhance the operation of the HVAC system under normal conditions to provide additional cooling capacity, improve air distribution and temperature control. The modifications will also supplement the cooling requirements for the entire Control Building Annex during Design Bases Accident (DBA) conditions.

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.



PLANT CHANGE/MODIFICATION 87-212-11

UNIT : 3/4
TURN OVER DATE : 08/29/91

EDG ENHANCEMENT - SITE PREPARATION

Summary:

This Engineering Package details the requirements for preliminary site work to be performed prior to the construction of a new diesel generator building located between Turkey Point Units 2 and 3, just west of the water treatment area. The new building contains two new diesel generators which will enhance the emergency power capability at Turkey Point.

The preliminary site work will include the permanent relocation of the Unit 3 containment access ramp, installation of a maintenance ramp, the installation of diesel oil interconnecting piping and electrical duct bank beneath the ramp, relocation of the diesel oil fill line in the ramp area, the removal of the maintenance building located between Units 2 and 3, the temporary relocation of the RCA fence, exploratory digging in the vicinity of the new building foundation excavation, relocation of the fire water piping, relocation of the screen refuse piping, relocation of the service air and water piping and the outdoor lighting in the vicinity of the new building and final site grading and drainage.

Safety Evaluation:

There are no accidents or malfunctions of a different type from the safety analysis created by this modification. This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.



PLANT CHANGE/MODIFICATION 87-257

UNIT : 3/4
TURN OVER DATE : 09/19/91

LOAD CENTER 4H, MCC 4D AND TRANSFER OF LOADS

Summary:

This Engineering Package (EP) encompasses the engineering/design for the installation of 480 Volt Load Center 4H and Motor Control Center (MCC) 4D in the new Electrical Equipment Room (refurbished Hot Machine Shop). This EP also includes the installation of new circuit breakers in 480 Volt Load Centers 4C and 4D to function as feeder breakers to Load Center 4H. The relocation of Charging Pump 4C motor feeder and controls from Load Center 4C to new Load Center 4H and the transferring of certain loads from existing MCC D (to be relabeled 3D via PC/M 87-258) to new MCC 4D are also included in this package.

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.

PLANT CHANGE/MODIFICATION 87-258

UNIT : 3/4
TURN OVER DATE : 09/18/91

LOAD CENTER 3H AND REPOWERING OF MCC D (3D)

Summary:

This Engineering Package (EP) encompasses the engineering/design for the installation of existing 480 Volt Load Center 3H in the new Electrical Equipment Room (refurbished Hot Machine Shop) and the repowering of existing Motor Control Center (MCC) D (to be relabeled 3D) from this new load center. This EP also includes the installation of new circuit breakers in 480 Volt Load Centers 3C and 3D to function as feeder breakers to Load Center 3H. The relocation of Charging Pump 3C motor feeder and controls from Load Center 3C to new Load Center 3H and the removal of the Telemand transfer operator and circuitry from existing MCC D (3D) are also included in this package.

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.

PLANT CHANGE/MODIFICATION 87-259-02

UNIT : 3/4
TURN OVER DATE : 08/18/91

EDG ENHANCEMENT - EDG BUILDING ADDITION

Summary:

This engineering Package (EP) details the requirements for construction of a new diesel generator and diesel oil storage facility located between Turkey Point Units 2 and 3, just west of the water treatment area. The new structures will contain two (2) diesel generator sets with diesel oil storage tanks and transfer pumps, and will be assigned to Unit 4. The existing diesel generator building will be assigned to Unit 3.

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.



PLANT CHANGE/MODIFICATION 87-260

UNIT : 4
TURN OVER DATE : 08/20/91

UNIT 4 EPS ENHANCEMENT -FIRE PROTECTION

Summary:

This Engineering Package (EP provides the engineering design and procurement support necessary for the installation of automatic fire protection sprinkler systems for the following rooms in the new Diesel Building:

- a) Diesel Generator/A Compressor Rooms;
- b) Diesel Oil Transfer Pump Rooms:

It also addresses connection of these systems to the plant fire service main.

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.



PLANT CHANGE/MODIFICATION 87-261-01

UNIT : 3/4
TURN OVER DATE : 08/18/91

NEW EDG BUILDING LIGHTING, FIRE PROTECTION, COMMUNICATIONS AND
HVAC

Summary:

This Engineering Package encompasses the design, procurement and installation of the lighting, fire detection, communications and HVAC systems for the new Emergency Diesel-Generator (EDG) building. Non-safety related systems are designed and installed such that there will be no impact on the operation of Safety Related equipment or systems. No systems are tied into the existing plant via this PC/M. All final tie-ins to the existing plant will be performed under PC/M 87-264.

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.

PLANT CHANGE/MODIFICATION 87-263

Page 1 of 2

UNIT : 3/4
TURN OVER DATE : 09/13/91

NEW EDGS INSTALLATION .

Summary:

This Engineering Package encompasses the design, procurement, fabrication, installation and pre-operational testing of two Emergency Diesel-Generators (EDGs) and the following, above ground systems and components:

A. Safety Related Systems and Components

- a. Diesel Oil Transfer Systems external to the engine skids,
- b. Air Starting Systems external to the engine skids,
- c. Diesel Engine Radiator and piping external to the engine skids,
- d. Diesel Engine Exhaust Systems,
- e. Two EDG Control Panels,
- f. Two 480V Motor Control Centers (4J and 4K)
- g. Two 125V dc Distribution Panels (4D35 and 4D36)
- h. Interconnecting safety related cable and raceway in the new EDG Building, and
- f. Fuel Oil Crosstie Isolation Valves.

B. Nonsafety Related Systems and Components

- a. Diesel Oil Truck Fill Connections,
- b. Diesel Oil Storage Tanks' Inventory Level and Temperature Instrumentation,
- c. Level Instrumentation for the Oily Waste collection Sump,
- d. Diesel Engine-driven Air Compressor Engine Exhaust Systems,
- e. Service Water Systems,
- f. Demineralized Water Systems,
- g. Interconnecting nonsafety related cable and raceway in the new EDG Building and
- f. Fuel Oil Crosstie Piping

There will be no interfaces with, and no modifications made to the existing plant systems except for the Service/Demineralized Water and the Diesel Oil Transfer Systems. All other tie-ins to the existing plant will be performed under PCM 87-264.

PLANT CHANGE/MODIFICATION 87-263

Page 2 of 2

NEW EDGS INSTALLATION

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.



PLANT CHANGE/MODIFICATION 87-264

Page 1 of 2

UNIT : 3/4
TURN OVER DATE : 10/17/91

EDG 3B/4B, EDG 3A/4A AND NEW EDG BUILDING TIE-INS

Summary:

This Engineering Package (EP) encompasses the engineering/design for the installation of new equipment as follows:

- a. 480V Motor Control Center (MCC) 3K (including relocation of existing EDG B (3B) auxiliaries from 480V MCCs 4A and 4B),
- b. Emergency Diesel-Generator (EDG) Bus Load Sequencers 3C23A-1, 3C23B-1, 4C23A-1 and 4C23B-1,
- c. New EDG lockout/reset pushbuttons, indicating lights and Watthour meters in the Control Room, and
- d. All interconnecting cabling and raceway for the above equipment (PCMs 87-212 and 87-259 provided the new EDG Building, EDG Building embedded conduits and new duct banks; exposed raceways in the new EDG Building were provided in PC/M 87-263).

Also included in the scope of this EP is the engineering/design required for:

- a. The electrical tie-ins of the following Emergency Power System (EPS) Enhancement Project PCMs:
 1. PC/M 87-260, Fire Protection System,
 2. PC/M 87-261, New EDG Building Lighting, Fire Detection, communications and HVAC, and
 3. PC/M 87-263, New EDGs installation (installation of new EDGs 4A and 4B and supporting systems and components).
- b. The modifications required to electrically reconfigure existing EDGs A (to be relabeled 3A) and B (to be relabeled 3B) to power loads associated with a given unit and certain loads which are shared between the units and their supporting systems, and the removal of the Telemand Operators, circuitry and alternate train ties associated with MCCs 3A and 4A.
- c. Revisions to administrative procedures associated with the mechanical tie-ins to the existing plant systems for the new EDG Building Fire Protection and Diesel Fuel Oil Systems provided via PC/Ms 87-260 and 87-263 respectively.



EDG 3B/4B, EDG 3A/4A AND NEW EDG BUILDING TIE-INS

Supplement 1 of this EP will install EDG Load Sequencers and provide the balance of the modifications, including tie-ins to the existing plant. Supplement 1 will also provide the EPS Enhancement testing of all tie-ins between the new EDGs and the existing plant systems.

Safety Evaluation:

The modifications provided by this EP do not have any adverse effect on the plant safety or operation, and do not constitute an unreviewed safety question. Required revisions to the Turkey Point Final Safety Analysis Report will be provided in a future Emergency Power System (EPS) PC/M Supplement. Changes to the Turkey Point Plant Technical Specifications for tie-ins to the existing plant will be provided in a separate EPS Enhancement Project licensing submittal. Tie-ins to the existing plant and removal of the alternate feeder/Telemand operators associated with MCCs 3A/4A require prior NRC approval. All other modifications may be implemented without prior NRC approval.

PLANT CHANGE/MODIFICATION 87-265-08

UNIT : 3/4
TURN OVER DATE : 09/16/91

SWING SWITCHGEAR 3D AND TRANSFER OF ICW AND CCW 3C PUMPS

Summary:

This Engineering Package (EP) encompasses the engineering/design for the installation of the new 4.16kV Switchgear 3D and dc Transfer Switch 3S75 in the new Emergency Diesel-Generator (EDG) Building, installation of dc Distribution Panels 3D01A and 4D01A in the DC Equipment Rooms of the Control Building, and all cabling/exposed raceway servicing this equipment. This package includes the relocation of the Component Cooling Water (CCW) 3C and Intake Cooling Water (ICW) 3C motor feeders and controls from breakers in existing 4.16kV Switchgear 3A and 3B to the new 4.16kV Switchgear 3D. Also included in this package are the modifications of Switchgear 3A and 3B circuit breakers 3AA17 and 3AB19, respectively, to function as feeder tie breakers to 4.16kV Switchgear 3D and modifications to Control Room Vertical Panel 3C04 for manual tie breaker operation and 4.16kV Switchgear lockout/reset control.

Safety Evaluation:

None of the modifications provided for in this EP have any adverse impact on plant safety or its operation. This modification does not constitute an unreviewed safety question. Required revisions to the Turkey Point Final Safety Analysis Report (FSAR) will be provided in PC/M 87-264. However, revisions to the Turkey Point Technical Specifications, as proposed in FPL letter L-90-68, will be required for plant operation (i.e., entry into MODE 6 following the dual unit outage) after implementation of these modifications. Therefore, with the Mode restriction placed in this EP, prior NRC approval was not required for the modifications.



PLANT CHANGE/MODIFICATION 87-266-08

UNIT : 3/4
TURN OVER DATE : 09/19/91

SWING SWITCHGEAR 4D AND TRANSFER OF ICW AND CCW 4C PUMPS

Summary:

This Engineering Package (EP) encompasses the engineering/design for the installation of the new 4.16kV Switchgear 4D and dc Transfer Switch 4S75 in the new Emergency Diesel-Generator (EDG) Building and all cabling/exposed raceway servicing this equipment. This package includes the relocation of the Component Cooling Water (CCW) 4C and Intake Cooling Water (ICW) 4C motor feeders and controls from breakers in existing 4.16kV Switchgear 4A and 4B to the new 4.16kV Switchgear 4D. Also included in this package are the modifications of Switchgear 4A and 4B circuit breakers 4 M 17 and 4AB19, respectively, to function as feeder tie breakers to 4.16kV Switchgear 4D and modifications to Control Room Vertical Panel 4C04 for manual tie breaker operation.

Safety Evaluation:

None of the modifications provided for in this EP have any adverse impact on plant safety or its operation. The modifications provided for in this EP do not constitute an unreviewed safety question. Required revisions to the Turkey Point Final Safety Analysis Report (FSAR) will be provided in PC/M 87-264. Changes to the Turkey Point Plant Technical Specifications required for tie-ins to the existing plant have been provided by Amendments 138 and 133 to the Turkey Point Technical Specifications for Units 3 and 4, respectively. Therefore, prior NRC approval was not required for this modification.

PLANT CHANGE/MODIFICATION 87-267

UNIT : 3/4
TURN OVER DATE : 08/20/91

STATION BATTERY CHARGERS INSTALLATION

Summary:

This Engineering Package (EP) encompasses the engineering, design, installation, and testing of the new additional Station Battery Chargers 3A2(3D02A) and 3B2(3D25A) in the new Electrical Equipment Room (refurbished Hot Machine Shop via PC/M 89-069) and the reflasher annunciator in the Control Room, including required interconnecting cables and raceway between the above equipment and associated components, at Turkey Point Plant Units 3 and 4. Also, this EP includes the engineering, design and installation necessary for the tie-ins of the above new equipment to the existing plant systems.

Safety Evaluation:

The modifications provided for in this EP do not have any adverse impact on plant safety or its operation, and do not constitute an unreviewed safety question. Required revisions to the Turkey Point Final Safety Analysis Report (FSAR) will be provided in a future Emergency Power System (EPS) Enhancement Project PC/M Supplement. Changes to the Turkey Point Plant Technical Specification for final tie-ins will be provided in a separate EPS Enhancement Project Licensing submittal. Tie-ins to the existing plant require prior NRC approval of the Plant Technical Specification changes. All other modifications provided for in this EP may be implemented, when coordinated with operations, and do not require prior NRC approval.

PLANT CHANGE/MODIFICATION 87-333-04

UNIT : 3/4
TURN OVER DATE : 02/25/92

WASTE STORAGE FACILITY

Summary:

This Engineering Package provides the design for the construction of a Waste Storage Facility for Turkey Point Units 3 & 4 that meets the requirements of Federal Regulation title 40 CFR 264.175. The Turkey Point Plant has been classified as a large generator of hazardous waste.

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.

PLANT CHANGE/MODIFICATION 88-077-03

UNIT : 3
TURN OVER DATE : 05/07/92

UNIT 3 RESIDUAL HEAT REMOVAL PUMPS MECHANICAL SEAL AND SEAL
COOLER REPLACEMENT

Summary:

This Engineering Package provides details for the replacement of the mechanical seals and the seal coolers on the Residual Heat Removal pumps. The existing seals have proven to have an unsatisfactory seal life. The new seals have an upgraded design, and the seal coolers have a larger heat removal capability, which should provide a longer operational life.

Safety Evaluation:

There are no accidents or malfunctions of a different type from the safety analysis created by this modification. This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.



..
..
.. PLANT CHANGE/MODIFICATION 88-164-02

UNIT : 4
TURN OVER DATE : 07/11/91

..
AFW NITROGEN BACKUP SYSTEM RELIEF VALVE INSTALLATION

Summary:

This Engineering Package (EP) provides the design for the replacement of existing Rupture Disc Assemblies on the AFW Nitrogen Backup Systems with dual mounted relief valves. In addition, this EP provides for the installation of six test valves located on each instrument air header to the AFW Flow Control Valves, between the isolation valve and the check valve for Unit 4.

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.

PLANT CHANGE/MODIFICATION 88-169

UNIT : 3/4
TURN OVER DATE : 10/09/91

REACTOR CAVITY HANDRAIL AND SAFETY CABLES

Summary:

This Engineering Package provides for the addition of safety handrails adjacent to the reactor cavity refueling pit at elevations 58'-0", and a safety cable system along the manipulator crane rails at elevation 58'-0".

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.



PLANT CHANGE/MODIFICATION 88-170-02

UNIT : 3
TURN OVER DATE : 03/13/92

UNIT 3 REPLACEMENT OF SPENT FUEL POOL BRIDGE CRANE

Summary:

This Engineering Package replaces the Turkey Point Unit 3 Spent Fuel Pool Bridge Crane. The new bridge crane will be functionally similar to the existing crane, with modifications to improve reliability, ease of operation, and maintenance.

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.



PLANT CHANGE/MODIFICATION 88-180

UNIT : 3
TURN OVER DATE : 08/20/91

UNIT 3 MOV THERMAL OVERLOAD HEATER SELECTION

Summary:

This Engineering Package (EP) provides the thermal overload heater size selection for fifty five (55) Safety Related and ten (10) Quality Related MOVs. This EP also documents the overload relay heater selection for MOV-6459A, B and C. This selection reflects the existing heater installation for these MOVs and does not involve any physical modifications.

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.



PLANT CHANGE/MODIFICATION 88-181

UNIT : 4
TURN OVER DATE : 08/22/91

UNIT 4 MOV THERMAL OVERLOAD HEATER SELECTION

Summary:

This Engineering Package provides the thermal overload heater size selection for fifty (50) Safety Related and ten (10) Quality Related MOVs. In addition, as part of this modification, the existing thermal overload relay arrangement for affected ITE starters, which consists of two obsolete ITE Model E20COL1 Relays, shall be revised using ITE (Telemecanique) Model E20C1L1 Relays (three each for 480V ac starters and two each for 125V dc starters), which are direct replacements for the existing obsolete relays.

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.

PLANT CHANGE/MODIFICATION 88-226

UNIT : 3/4
TURN OVER DATE : 10/31/91

UNDERGROUND DRAINAGE SYSTEM SECURITY BARRIERS

Summary:

This Engineering Package provides for the addition of security barriers for thirteen openings (manholes, catch basins, and pipe ends) in the plant underground drainage systems. The barriers will secure the drainage systems against potential unauthorized access across the Protected Area (PA) and Vital Area (VA) boundaries through the plant drainage systems.

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.

PLANT CHANGE/MODIFICATION 88-228

UNIT : 3/4
TURN OVER DATE : 11/27/91

SECURITY PERIMETER FENCE MODIFICATIONS

Summary:

This Engineering Package provides the design for increasing the height of the security perimeter fence at a location south of the Units 1 and 2 intake canal and at another location south of the Units 3 and 4 intake canal.

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.



PLANT CHANGE/MODIFICATION 88-234

UNIT : 4
TURN OVER DATE : 10/04/91

RCS HOT AND COLD LEG BYPASS LOOP RTD REPLACEMENT

Summary:

This Engineering Package provides the design for replacement of the RCS Hot and Cold Leg Bypass Loop RTDs. This replacement is necessary as the existing safety related RTDs are reaching the end of their qualified life. Replacement of the RCS Bypass Loop RTDs will require a unit outage due to the protective function of the RTDs. This will ensure that plant operation, safety and availability are not affected during installation.

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.

PLANT CHANGE/MODIFICATION 88-245

UNIT : 4
TURN OVER DATE : 09/06/91

MSIV AIR ACCUMULATOR SYSTEM

Summary:

This Engineering Package provides the details for the modifications to the Main Steam Isolation Valve (MSIV) Air Accumulator System. This modification provides a dedicated safety related air reserve for each MSIV and will ensure that each MSIV will close in five seconds or less upon receipt of a closure signal coincident with the most limiting process conditions for valve closure. In addition, the modification ensures MSIV closure can be maintained for a minimum of one hour without the need for operator action independent of the availability of instrument air.

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.



PLANT CHANGE/MODIFICATION 88-265-01

UNIT : 3/4
TURN OVER DATE : 06/25/92

OIL RETENTION BASIN FOR EMERGENCY FIRE PUMP DIESEL OIL STORAGE
TANK

Summary:

This Engineering Package provides the design for installation of an oil retention basin for the Emergency Fire Pump Diesel Oil Storage Tank. The function of the oil retention basin is to retain any potential diesel oil spill from the existing 550 gallon storage tank to assure compliance with environmental requirements.

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.

PLANT CHANGE/MODIFICATION 88-289

UNIT : 3
TURN OVER DATE : 03/29/90

RCS HOT AND COLD LEG BYPASS LOOP RTD REPLACEMENT

Summary:

This Engineering Package provides the design for replacement of the RCS Hot and Cold Leg Bypass Loop RTDs. This replacement is necessary as the existing safety related RTDs are reaching the end of their qualified life. This modification replaces twelve RTDs, two per loop. Six of these RTDs are used for normal operation while the other six serve as installed spares. Replacement of the RCS Bypass Loop RTDs will require a unit outage due to the protective function of the RTDs. This will ensure that plant operation, safety and availability are not affected during installation.

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.

PLANT CHANGE/MODIFICATION 88-319-05

UNIT : 3/4
TURN OVER DATE : 10/25/91

MODIFICATIONS TO SUPPORT SPENT FUEL POOL RERACKING

Summary:

This Engineering Package (EP) provides the designs, documentation, references, and instructions for several modifications necessary to support the installation of the new high density spent fuel storage racks in the Unit 4 Spent Fuel Pool. In addition, this EP addresses the modifications required to accommodate two additional re-rack tools on both the Unit 3 and Unit 4 refueling tool storage racks, as well as the modifications to resolve any interferences between the tool racks and storage locations directly below the tools.

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.



PLANT CHANGE/MODIFICATION 88-350-02

UNIT : 3/4
TURN OVER DATE : 08/20/91

STATION BATTERY CHARGERS REPLACEMENT

Summary:

This Engineering Package encompasses the design, installation and testing of six station battery chargers to replace the existing safety related Station Battery Chargers 3A (3D02, to be relabeled 3B1), 3B (3D25, to be relabeled 3B1), 3S (D02), to be relabeled 4B2 (4D02A), 4A (4D25, to be relabeled 4A1), 4B (4D02, to be relabeled 4B1) and 4S (D26), to be relabeled 4A2 (4D25A), with new station battery chargers, and new 125V DC Distribution Panels 3D23A and 4D23A, including any required interconnecting raceways/cables between the above equipment and associated equipment, at the Turkey Point Plant Units 3 and 4. Also, the Station Battery Chargers 3S (4B2) and 4S (4A2) are relocated to the new Electrical Equipment Room (refurbished hot Machine Shop via PC/M 89-069), in lieu of their present location in the DC Equipment Rooms 4B and 4A, respectively.

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.

PLANT CHANGE/MODIFICATION 88-363-02

UNIT : 3/4
TURN OVER DATE : 12/03/91

SECURITY LIGHTING

Summary:

This Engineering Package encompasses the engineering and design of a Security Lighting System. This effort includes the procurement and installation of the following equipment:

- a. High pressure sodium floodlights
- b. High mast poles
- c. Service hoist for high mast poles
- d. Aircraft warning lights
- e. Photoelectric controls
- f. Supplemental lighting

With the exception of the incorporation of the existing lighting atop the Nuclear Administration Building and the Nuclear Maintenance Building, and wallighter type luminaires, there are no permanent interfaces with and no modifications have been made to existing plant systems.

Safety Evaluation:

This modification did not have any adverse effect on the plant safety or operation. This modification did not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.

PLANT CHANGE/MODIFICATION 88-364-10

UNIT : 3/4
TURN OVER DATE : 12/19/91

SECURITY SYSTEM UPGRADE PROJECT - FENCE AND INTRUSION DETECTION
SYSTEM

Summary:

This Engineering Package (EP) encompasses the design, engineering and installation of the following electrical equipment or structures for the Security System Upgrade:

- a. Microwave, infrared, barrier net and ported coaxial cable intrusion detection systems
- b. Metal detector, x-ray search equipment, explosive detector, turnstiles and radiation monitor for the new Water Treatment Area Access Control Guardhouse
- c. Perimeter and nuisance fences, removable nuisance chain
- d. Grating partition between nuclear and fossil units

In addition, this EP encompasses the design, installation and/or modification the following equipment or structures:

- a. Installation of cable and conduit between the local Intrusion Detection System (IDS) multiplexers and the intrusion detection equipment
- b. Installation of discharge canal elevated platform
- c. Installation of new Water Treatment Access Control Guardhouse
- d. Modifications to warehouse on the northwest perimeter
- e. Removal of Chlorinator Building
- f. Regrading of existing slopes along perimeter
- g. Modifications to Plant Fire Main (piping associated with 10CFR50 Appendix R Fire Protection System)

Safety Evaluation:

The modification described herein shall be designed and installed such that there will be no impact on the operation of safety related equipment or systems. Thus, the modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.

PLANT CHANGE/MODIFICATION 88-365-05

UNIT : 3/4
TURN OVER DATE : 12/19/91

SECURITY UPGRADE PROJECT SECURITY COMPUTER AND AUXILIARY
EQUIPMENT

Summary:

The Turkey Point Security System is being enhanced with a new improved installation using high mast high pressure sodium floodlights, improved fencing and intrusion detection system, closed circuit television, new computer systems, and a backup power supply. As part of this change, Units 1 and 2 will be removed from the existing Security Area.

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.

PLANT CHANGE/MODIFICATION 88-416-01

UNIT : 3/4
TURN OVER DATE : 09/30/91

ADDITION OF SPARE STATION BATTERY

Summary:

This Engineering Package provides a permanent spare station battery and associated switching system. This modification satisfies INPO Recommendation TS 5.1 to allow for the replacement of any of the four existing vital station batteries (3A, 3B, 4A or 4B) for capacity testing in accordance with IEEE Standard 450. This modification is an enhancement enabling the plant to perform scheduled or unscheduled battery maintenance and testing on the vital station batteries without entering into a Limiting Condition of Operation (LCO).

Safety Evaluation:

Revisions to the Turkey Point Technical Specifications as proposed on the FPL letter L-90-68, will be required for Plant operation (i.e., before entry into Mode 6 following the dual unit outage) after implementation of these modifications. An administrative requirement prevents the unit from entering Mode 6 until this license amendment has been issued by the NRC. With this restriction in mind, this evaluation has shown that this modification does not constitute an unreviewed safety question and has no adverse effects on plant safety and operation. Therefore, prior NRC approval was not required for this modification with this Mode restriction.

PLANT CHANGE/MODIFICATION 88-418-07

UNIT : 3
TURN OVER DATE : 09/06/91

SECURITY SYSTEM UPGRADE PROJECT UNIT 3 VITAL AREA BARRIERS

Summary:

This Engineering Package (EP) provides the engineering, design and installation of upgraded Vital Area Barriers (VAB) in support of the Security System Upgrade Project at Turkey Point Unit 3. The scope is to replace the presently existing chain link fence barriers with upgraded hardened Vital Area Barriers. The actual areas that will be modified by the EP are as follows:

- a. Main Steam Platform
- b. Operating Deck Enclosure
- c. AFW Steam Supply Platform
- d. Condensate Storage Tank
- e. Feedwater Platform
- f. AFW Isolation Valves Area
- g. AFW Pump Enclosure

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.

PLANT CHANGE/MODIFICATION 88-419-085

UNIT : 4
TURN OVER DATE : 11/12/91

SECURITY SYSTEM UPGRADE PROJECT UNIT 4 VITAL AREA BARRIERS

Summary:

This Engineering Package (EP) provides the engineering, design and installation of upgraded Vital Area Barriers (VAB) in support of the Security System Upgrade Project at Turkey Point Unit 4. The scope is to replace the presently existing chain link fence barriers with upgraded hardened Vital Area Barriers. The actual areas that will be modified by the EP are as follows:

- a. Main Steam Platform
- b. Operating Deck Enclosure
- c. AFW Steam Supply Platform
- d. Condensate Storage Tank
- e. Feedwater Platform
- f. AFW Isolation Valves Area
- g. AFW Pump Enclosure

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.

UNIT : 3
TURN OVER DATE : 08/08/91

UNIT 3 MOV TORQUE AND LIMIT SWITCH SETTINGS

Summary:

This Engineering Package (EP) provides the following for the Motor Operated Valves (MOVs) at Unit 3 which are included in the scope of this package:

- a) MOV switch setting policy for MOV torque and limit switches, including the use of stem thrust measurement and signature analysis techniques to determine and/or verify switch settings and including increased torque bypass switch settings to improve operability.
- b) Justification for the use of thrust measurement and signature analysis techniques as an enhancement with respect to the use of published torque settings.
- c) Revised MOV limit switch contact assignments and associated wiring changes to ensure that Control Room valve position indication is maintained following revised torque bypass switch settings included in (a) above.

The following valves are within the scope of this EP:

VALVE	SYSTEM
MOV-3-750	Residual Heat Removal (RHR)
MOV-3-751	Residual Heat Removal
MOV-3-535	Reactor Coolant System (RCS)
MOV-3-536	Reactor Coolant System
MOV-3-865A, B, C	Safety Injection (SI)
MOV-3-744A, B	Residual Heat Removal
MOV-3-866A, B	Safety Injection
MOV-3-6386	Chemical Volume and Control System (CVCS)

These MOV operators are classified as Class 1E and perform a safety related function within the RHR, SI, CVCS and RCS systems and/or provide containment isolation.



PLANT CHANGE/MODIFICATION 88-480
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UNIT 3 MOV TORQUE AND LIMIT SWITH SETTINGS

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.



PLANT CHANGE/MODIFICATION 88-481-03

UNIT : 4
TURN OVER DATE : 07/17/91

UNIT 4 MOV TORQUE AND LIMIT SWITCH ENHANCEMENTS

Summary:

This Engineering Package (EP) provides the following for the Motor Operated Valves (MOV) at Unit 4 which are included in the scope of this package:

- a) MOV switch setting policy for MOV torque and limit switches, including the use of stem thrust measurement and signature analysis techniques to determine and/or verify switch settings and including increased torque bypass switch settings to improve operability.
- b) Justification for the use of thrust measurement and signature analysis techniques as an enhancement with respect to the use of published torque settings.
- c) Revised MOV limit switch contact assignments and associated wiring changes to ensure that Control Room valve position indication is maintained following revised torque bypass switch settings included in (a) above.

The following valves are within the scope of this EP:

VALVE	SYSTEM
MOV-4-750	Residual Heat Removal (RHR)
MOV-4-751	Residual Heat Removal

These MOVs are required for safe plant shutdown and LOCA condition, are classified as Class 1E, are environmentally and seismically qualified and perform a safety related function.

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.



PLANT CHANGE/MODIFICATION 88-492-03

UNIT : 3/4
TURN OVER DATE : 09/20/91

MACHINE SHOP

Summary:

This Engineering Package provides the design for the construction of a Machine Shop at Turkey Point Units 3 & 4. The Machine Shop has been designated to contain the new Central Alarm Station (CAS) which will occupy an approximate 25' by 25' area of the building.

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.



PLANT CHANGE/MODIFICATION 88-503-04

UNIT : 3/4
TURN OVER DATE : 09/24/91

NUCLEAR ENTRANCE BUILDING

Summary:

This Engineering Package provides for the design and specifications for the construction of a new Nuclear Entrance Building (NEB) for Turkey Point Units 3 and 4. This facility will support efficient plant access and will provide a centralized security operations facility. This building is part of the Security Upgrade Project.

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.



PLANT CHANGE/MODIFICATION 88-517-02

UNIT : 3
TURN OVER DATE : 07/10/91

UNIT 3 ATMOSPHERIC DUMP VALVES AND STOP VALVE REPLACEMENT

Summary:

This Engineering Package provides the details for the replacement of Atmospheric Dump Valves (ADV) and their coordinating stop valves in the Main Steam System due to steam leakage through the existing valves.

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.



PLANT CHANGE/MODIFICATION 88-535-08

UNIT : 4
TURN OVER DATE : 10/10/91

UNIT 4 PRESSURIZER PORV AIR AND NITROGEN SUPPLY TUBING
ENHANCEMENT

Summary:

This Engineering Package provides for modification of the existing PORV actuator covers and replacement of the pressure regulators and indicators, relief valves with rupture discs, and increases the size of the solenoid valves, tubing, in-line valves, and fittings for the air and nitrogen supply lines to enhance the opening times for the PORVs. In addition, an accumulator will be installed in the PORV actuation supply system. A bypass with an in-line check valve will also be installed around the solenoid valve closest to the PORV, to assure that a single failed solenoid valve will not prevent a PORV from closing. In addition, all manual valves will be relocated from inside the pressurizer cubicle to outside the pressurizer cubicle to enhance access and system maintenance.

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.



PLANT CHANGE/MODIFICATION 88-541-01

UNIT : 4
TURN OVER DATE : 10/25/91

UNIT 4 ATMOSPHERIC DUMP VALVES AND STOP VALVE REPLACEMENT

Summary:

This Engineering Package provides details for the replacement of Atmospheric Dump Valves (ADV) and their corresponding Stop Valves in the Main Steam System due to steam leakage through the existing valves.

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.

PLANT CHANGE/MODIFICATION 88-548

UNIT : 4
TURN OVER DATE : 09/20/91

UNIT 4 ACCUMULATORS LEVEL TRANSMITTER ACCESS PLATFORMS

Summary:

This Engineering Package provides details for the modifications which install access platforms and provide additional lighting for maintenance and calibration of the accumulator level transmitters at their new location.

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.

PLANT CHANGE/MODIFICATION 88-580-02

UNIT : 3/4
TURN OVER DATE : 09/26/91

DRAINAGE AND GRADING ENHANCEMENT AT SE CORNER

Summary:

This Engineering Package details the requirements to construct several new buildings and paved vehicle surfaces at the Southeast (SE) corner of Units 3 and 4. The existing ground elevation will be raised to Elevation 17'-0" \pm with structural fill. The structural fill will be capable of supporting future surface construction.

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.

PLANT CHANGE/MODIFICATION 88-582-01

UNIT : 3
TURN OVER DATE : 08/22/91

UNIT 3 RPI INVERTER UPGRADE

Summary:

This Engineering Package provides the design to upgrade the Rod Position Indication System power supply. This modification is necessary to improve availability of replacement parts since the manufacturer of the existing inverter can no longer provide spare parts.

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.

PLANT CHANGE/MODIFICATION 88-583-02

UNIT : 4
TURN OVER DATE : 09/17/91

UNIT 4 RPI INVERTER UPGRADE

Summary:

This Engineering Package provides the design to upgrade the Rod Position Indication System power supply. This modification is necessary to improve availability of replacement parts since the manufacturer of the existing inverter can no longer provide spare parts.

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.

PLANT CHANGE/MODIFICATION 89-059-01

UNIT : 4
TURN OVER DATE : 05/14/92

UNIT 4 SUPPORT MODIFICATION FOR VALVES FCV-4-605 AND HCV-4-758 ON
THE RESIDUAL HEAT REMOVAL SYSTEM, OUTSIDE CONTAINMENT

Summary:

This Engineering Package details the modifications to the actuator supports for valves FCV-4-605 and HCV-4-758. This portion of the RHR System, between the RHR pumps, the RHR Heat Exchangers, and the containment penetration is Safety Related, Quality Group B. The pipe support modification contained in this DEEP are accordingly considered Safety Related.

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.

PLANT CHANGE/MODIFICATION 89-069

UNIT : 3/4
TURN OVER DATE : 09/04/91

NEW ELECTRICAL EQUIPMENT ROOM

Summary:

This Engineering Package provides the design and construction requirements to convert the existing hot machine shop in the Auxiliary Building into a New Electrical Equipment Room, which will also house a new Battery Room.

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.



PLANT CHANGE/MODIFICATION 89-070-06

UNIT : 3
TURN OVER DATE : 09/07/91

UNIT 3 INSTALLATION OF THE CHEMICAL INJECTION SYSTEM FOR THE
INTAKE COOLING WATER SYSTEM

Summary:

This Engineering Package installs injection nozzles in the Intake Cooling Water (ICW) piping and, chemical feedlines in the Component Cooling Water (CCW) heat exchanger room to support a test program to be conducted by the site Technical Department. This program will establish the effectiveness of preselected chemical agents in maintaining CCW heat exchanger thermal capacity.

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.

PLANT CHANGE/MODIFICATION 89-101-01

UNIT : 3/4
TURN OVER DATE : 08/14/91

SWITCHYARD DUCT BANK ADDITION TO FLY-OVER STRUCTURE

Summary:

This Engineering Package provides for the design and instructions for the installation of an electrical duct bank system, associated manhole and electrical raceway system and the fly-over structure raceways. This will allow additional connections from the switchyard to the existing plant raceway system.

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.



PLANT CHANGE/MODIFICATION 89-103-01

UNIT : 3
TURN OVER DATE : 04/09/91

UNIT 3 CONDENSATE OXYGEN ANALYZER REPLACEMENT

Summary:

This Engineering Package provides for the replacement of the existing dissolved oxygen (D.O.) analyzer. The Condensate D.O. analyzer is utilized to determine the concentration of dissolved oxygen in the condensate to assure that the steam generators, turbines and other plant equipment are subjected to minimal corrosion and deterioration caused by dissolved oxygen.

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.

PLANT CHANGE/MODIFICATION 89-104-01

UNIT : 3
TURN OVER DATE : 04/09/91

UNIT 4 CONDENSATE OXYGEN ANALYZER REPLACEMENT

Summary:

This Engineering Package provides for the replacement of the existing dissolved oxygen (D.O.) analyzer. The Condensate D.O. analyzer is utilized to determine the concentration of dissolved oxygen in the condensate to assure that the steam generators, turbines and other plant equipment are subjected to minimal corrosion and deterioration due to dissolved oxygen.

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.

PLANT CHANGE/MODIFICATION 89-141-02

UNIT : 3/4
TURN OVER DATE : 10/05/90

POST ACCIDENT SAMPLING SYSTEM CHLORIDE ANALYZER REPLACEMENT

Summary:

This Engineering Package provides for the installation of a replacement Post Accident Sampling System (PASS) Chloride Analyzer for Units 3 & 4. The existing PASS Chloride Analyzer is unreliable.

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.



PLANT CHANGE/MODIFICATION 89-159-06

UNIT : 3/4
TURN OVER DATE : 08/21/91

FLOOD PROTECTION FOR EDG FUEL OIL TRANSFER PUMPS

Summary:

This Engineering Package (EP) modifies the foundations for the two Emergency Diesel Generator (EDG) fuel oil transfer pumps (3P10 and 4P10). The existing EDG fuel oil transfer pumps are located outside the flood protection barriers in an area which may experience a postulated flood to elevation 20'-0" above Mean Low Water (MLW) per the UFSAR. The top of each existing EDG fuel oil transfer pump foundation was surveyed for the as-built condition and it was determined that the top of concrete elevation is 18.3 feet. This EP will raise the top of each pump foundation to elevation 20'-0" above MLW.

Safety Evaluation:

The modifications required by this Engineering Package affect EDG system operability but was completed during the outage, prior to entering Mode 6. All work is governed by the conditions and restrictions imposed by the Safety Evaluation. With these restrictions in mind, this modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.



PLANT CHANGE/MODIFICATION 89-164-02

UNIT : 3/4
TURN OVER DATE : 09/09/91

CENTRAL RECEIVING FACILITY

Summary:

This Engineering Package provides the necessary design, specifications and instructions for the construction of a new Central Receiving Facility (CRF) for Turkey Point (PTP) Units 1, 2, 3, & 4.

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.

PLANT CHANGE/MODIFICATION 89-254-04

UNIT : 3/4
TURN OVER DATE : 09/17/91

REPLACEMENT OF GLAND FLANGES FOR VARIOUS ROCKWELL VALVES

Summary:

This Engineering Package (EP) evaluates replacement of the existing gland flange on various manual valves which are Rockwell Edwards Figure 3624 F316J, Westinghouse tag number 3/4-T58, with a replacement gland flange and plate washer. The two-piece configuration will facilitate replacement of the gland flange without removal of the yoke assembly. The affected valves are safety related therefore this EP is classified as safety related.

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.

PLANT CHANGE/MODIFICATION 89-272-01

UNIT : 3
TURN OVER DATE : 12/31/91

SPENT FUEL PIT COOLING SYSTEM ADDITIONAL SEISMIC UPGRADES

Summary:

This Engineering Package (EP) provides the design for seismic upgrades to the Spent Fuel Pit (SFP) Cooling System in addition to seismic upgrades implemented per PC/M 85-147. The seismic upgrades provided by this EP include replacement of the SFP Cooling Pumps' suction and discharge pressure indicators with seismically qualified pressure indicators and replacement of the flow indicator installed for in-service testing (IST) of the SFP Cooling Pumps with a seismically qualified flow indicator. In addition, this EP includes enhancement of the SFP Cooling Pumps' motor casing heater circuits.

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.

PLANT CHANGE/MODIFICATION 89-319

UNIT : 3
TURN OVER DATE : 09/06/91

UNIT 3 CHARMS TO ERDADS INPUT MODIFICATION

Summary:

This Engineering Package (EP) provides details for the modification of Containment High Area Radiation Monitoring System (CHARMS) circuitry to provide a more accurate, adjustable input to the Emergency Response Data Acquisition and Display System (ERDADS) system. This modification consists of adding a precision resistor, wiring changes, an isolation module change, and software changes. This EP is Safety Related because it modifies class 1E circuitry required for a Reg. Guide 1.97 Category 1 variable.

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.



PLANT CHANGE/MODIFICATION 89-320-01

UNIT : 4
TURN OVER DATE : 09/09/91

UNIT 4 CHARMS TO ERDADS INPUT MODIFICATION

Summary:

This Engineering Package (EP) provides details for the modification of Containment High Area Radiation Monitoring System (CHARMS) circuitry to provide a more accurate, adjustable input to the Emergency Response Data Acquisition and Display System (ERDADS) system. This modification consists of adding a precision resistor, wiring changes, an isolation module change, and software changes. This EP is Safety Related because it modifies class 1E circuitry required for a Reg. Guide 1.97 Category 1 variable.

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.



PLANT CHANGE/MODIFICATION 89-332

UNIT : 3
TURN OVER DATE : 08/21/91

GENERIC LETTER 88-17 LOSS OF DECAY HEAT REMOVAL PROGRAMMED
ENHANCEMENT - RCS REDUNDANT LEVEL MONITORS

Summary:

This Engineering Package (EP) provides the design for the replacement of the existing pressure transmitter, LT-3-6421, with a differential pressure transmitter referenced back to the Pressurizer thus providing a more reliable RCS level indication. To lessen ALARA concerns during routine maintenance, new Differential Pressure Transmitter LT-3-6421 will be located outside the bio-shield wall in lieu of the present location below 'A' Cold Leg at Elevation 14 ft - 0 in. inside the bio-shield wall. In addition, this EP provides the design for the installation of a new differential pressure transmitter (LT-3-6423) independent of and redundant to LT-3-6421. The new differential pressure transmitters will provide signals to a new digital dual bargraph Control Room indicating switch. Adjustable setpoint capability and common annunciation in Containment (horns) and the Control Room for both RCS level sensing loops and an output to the Safety Parameters Display System (SPDS) for Loop 6423 are included in the design.

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.



PLANT CHANGE/MODIFICATION 89-333

UNIT : 4
TURN OVER DATE : 09/19/91

GENERIC LETTER 88-17 LOSS OF DECAY HEAT REMOVAL PROGRAMMED
ENHANCEMENT - RCS REDUNDANT LEVEL MONITORS

Summary:

This Engineering Package (EP) provides the design for the replacement of the existing pressure transmitter, LT-4-6421, with a differential pressure transmitter referenced back to the Pressurizer thus providing a more reliable RCS level indication. To lessen ALARA concerns during routine maintenance, new Differential Pressure Transmitter LT-4-6421 will be located outside the bio-shield wall in lieu of the present location below 'A' Cold Leg at Elevation 14 ft - 0 in. inside the bio-shield wall. In addition, this EP provides the design for the installation of a new differential pressure transmitter (LT-4-6423) independent of and redundant to LT-4-6421. The new differential pressure transmitters will provide signals to a new digital dual bargraph Control Room indicating switch. Adjustable setpoint capability and common annunciation in Containment (horns) and the Control Room for both RCS level sensing loops and an output to the Safety Parameters Display System (SPDS) for Loop 6423 are included in the design.

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.

PLANT CHANGE/MODIFICATION 89-336-01

UNIT : 4
TURN OVER DATE : 07/31/91

UNIT 4 CONTAINMENT ELECTRICAL PENETRATION REPLACEMENT AND
INSTALLATION

Summary:

This Engineering Package (EP) provides for the installation of a new spare Unit 4 containment Electrical Penetration Assembly (EPA), to make up for the shortage of spare conductors resulting from extensive modifications for compliance with Appendix R requirements. Additionally, this EP provides for the replacement of one EPA that experienced grounding problems during start-up tests. The replacement EPA will also have additional spare conductors.

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.

PLANT CHANGE/MODIFICATION 89-378

UNIT : 3
TURN OVER DATE : 07/14/91

UNIT 3 MODIFICATIONS OF INSTRUMENT LOOPS

Summary:

This Engineering Package provides for the wiring modification associated with Process and Control System cabinets which have non-isolated inputs going to the Safety Parameter Display System (SPDS). These non-isolated inputs to SPDS have been attributed with causing excessive loading on the primary instrument loop by a common mode voltage problem. The following SPDS inputs are required per Regulatory Guide 1.97 and are being modified to minimize common mode voltage exceeding the recommended limit of +10 volts to earth ground potential.

Loop No.	Description
LT-3-470	Pressurizer Relief Tank Level
LT-3-115	Volume Control Tank Level
LT-3-112	Volume Control Tank Level

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.



PLANT CHANGE/MODIFICATION 89-384

UNIT : 3/4
TURN OVER DATE : 10/25/91

FIRE BARRIER, REACH ROD PENETRATION UPGRADE

Summary:

This Engineering Package provides the details for modification of twenty-one valve reach rod penetrations on Elevation 18'-0" main walkways of the Auxiliary Building found to be without the approved fire rated penetration in place as required by Appendix R. The twenty-one (21) reach rod penetration modifications will provide (a) the required fire barriers mandated by Appendix R, and (b) enhanced protective shielding to isolate the hallway from radioactive piping below the floor.

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.



PLANT CHANGE/MODIFICATION 89-451

UNIT : 3
TURN OVER DATE : 02/20/92

UNIT 3 REMOVAL OF TEMPERATURE SWITCHES TS-2107 AND TS-2108

Summary:

This Engineering Package provides the justification and instructions for the removal of temperature switches TS-2107 and TS-2108. These temperature switches are presently installed in thermowells on the Intake Cooling Water discharge piping from the Turbine Plant Cooling Water heat exchangers (TS-2107) and the Component Cooling Water heat exchangers (TS-2108). The ICW discharge piping from these heat exchangers discharge into the Cooling Canal System (Ultimate Heat Sink).

Safety Evaluation:

The implementation of this modification does not change the functional or operational requirements (as amended by this Engineering Package) of the existing system. This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.



PLANT CHANGE/MODIFICATION 89-457-02

UNIT : 3
TURN OVER DATE : 08/17/91

UNIT 3 GENERIC LETTER 88-17 LOSS OF DECAY HEAT REMOVAL PROGRAMMED
ENHANCEMENT - RESIDUAL HEAT REMOVAL FLOW

Summary:

This Engineering Package provides the design for improved RHR flow indication including an adjustable flow alarm in addition to the current fixed low flow alarm. RHR flow indication is currently provided in the Control Room on an indicator with a nonlinear scale. The present indicator will be changed to a digital bargraph indicator programmed for square root extraction, thus providing more accurate indication in the lower range. The hardware required to accomplish this change includes the addition of the digital bargraph indicator (which will convert the signal to a linear output) and a dual comparator module in place of the present single comparator module for additional alarm capability. Further, the Control Room annunciator window will be engraved to reflect the new alarm conditions.

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.

PLANT CHANGE/MODIFICATION 89-462

UNIT : 3/4
TURN OVER DATE : 08/23/91

AREA RADIATION MONITORING SYSTEM (ARMS) UPGRADE

Summary:

This Engineering Package provides the design to replace all 24 channels of the existing Area Radiation Monitoring System (ARMS) to comply with commitments made in Nuclear Licensing letter L-88-290 and FPL's response to Regulatory Guide (RG) 1.97, Rev. 3. All 24 channels are connected to the SPDS/SAS (Safety Parameter Display System/Safety Assessment System).

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.

PLANT CHANGE/MODIFICATION 89-532-03

UNIT : 3
TURN OVER DATE : 08/08/91

UNIT 3 PCB LOAD CENTER TRANSFORMER REPLACEMENT

Summary:

This Engineering Package provides the design to replace the four safety related load center transformers at Turkey Point Unit 3. The existing transformers are being replaced since their insulating liquid contains polychlorinated biphenyls (PCBs) which are considered to be toxic environmental contaminants.

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.

PLANT CHANGE/MODIFICATION 89-533-03

UNIT : 4
TURN OVER DATE : 08/17/91

UNIT 4 PCB LOAD CENTER TRANSFORMER REPLACEMENT

Summary:

This Engineering Package provides the design to replace the four safety related load center transformers at Turkey Point Unit 4. The existing transformers are being replaced since their insulating liquid contains polychlorinated biphenyls (PCBs) which are considered to be toxic environmental contaminants.

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.

PLANT CHANGE/MODIFICATION 89-549-02

UNIT : 3
TURN OVER DATE : 08/07/91

UNIT 3 REPAIR AND MODIFICATION OF INTAKE STRUCTURE

Summary:

This Engineering Package (EP) provides the necessary design, documentation, references and instructions to repair and modify degraded concrete slabs located in all Unit 3 Intake Structure bays. This EP provides the modifications required to restore the concrete beams supporting Unit 3 ICW Pumps and Screen Wash Pumps to a condition that meets the original design intent of the slab.

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.



PLANT CHANGE/MODIFICATION 89-565-01

UNIT : 3
TURN OVER DATE : 07/08/91

UNIT 3 ALTERNATE PRESSURIZER PORV AIR/NITROGEN SUPPLY TUBING
ENHANCEMENT

Summary:

To improve system reliability, the existing Copes-Vulcan Power Operated Relief Valve (PORV) air/nitrogen actuation system will be modified. This Engineering Package provides for replacement of the existing PORV actuator covers, replacement of the pressure regulators, pressure indicators, relief valves, and increasing the size of the solenoid valves, tubing, in-line valves, hoses and fittings for the air and nitrogen supply lines to enhance the opening stroke times for the PORVs. Needle valves with locking devices, which will be administratively controlled, will be added to further enhance adjustment of opening and closing times. Phenolic handles shall be installed in place of the standard tee handles to facilitate the locking devices. In-line filters will be added at the inlet to the nitrogen pressure regulators.

A bypass line with check valve will be installed around the solenoid valve closest to the PORV, to assure that a single failed solenoid valve will not prevent a PORV from closing. In addition, manual valves will be relocated from inside the pressurizer cubicle to outside the pressurizer cubicle, as feasible, to enhance accessibility and maintainability and reduce operating personnel radiation exposure.

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.

PLANT CHANGE/MODIFICATION 89-585

UNIT : 3
TURN OVER DATE : 08/22/91

UNIT 3 MG-6 SAFEGUARD ACTUATION RELAY REPLACEMENT

Summary:

This PC/M was issued to replace the existing safeguard MG-6 relays, style number 289B363A11 (12 relays are installed in Unit 3) with new MG-6 relays, style number 1163828. The old style MG-6 relays have been reported to have latching problems at Indian Point 2 Plant which could be attributed to the relay's internal cutoff contact. The old style relay (now obsolete) is being replaced with the new style relay, manufactured without the coil cutoff contact, to preclude any latching failure concerns associated with the coil cutoff contact.

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.

PLANT CHANGE/MODIFICATION 89-586

UNIT : 4
TURN OVER DATE : 08/30/91

UNIT 4 MG-6 SAFEGUARD ACTUATION RELAY REPLACEMENT

Summary:

This PC/M was issued to replace the existing safeguard MG-6 relays, style number 289B363A11 (12 relays are installed in Unit 4) with new MG-6 relays, style number 1163828. The old style MG-6 relays have been reported to have latching problems at Indian Point 2 Plant which could be attributed to the relay's internal cutoff contact. The old style relay (now obsolete) is being replaced with the new style relay, manufactured without the coil cutoff contact, to preclude any latching failure concerns associated with the coil cutoff contact.

Safety Evaluation:

This modification has been evaluated in accordance with 10 CFR 50.59, and found not to give rise to any unreviewed safety questions, and to not affect the plant Technical Specifications. Consequently, implementation of this PC/M does not require prior NRC approval.

PLANT CHANGE/MODIFICATION 90-047

UNIT : 3/4
TURN OVER DATE : 03/05/92

MATERIAL SUBSTITUTION-VALVE PARTS FOR FCV-3/4-479, 489, AND 499

Summary:

This Engineering Package provides justification to incorporate alternate material specifications for two parts (Plug and Cage) which are components of control valves installed in the Feedwater System. The subject control valves are the bypass valves for the main feedwater control valves. The alternate material specifications were initiated by the valve vendor (Copes-Vulcan) to reduce the potential for cracking.

Safety Evaluation:

This modification did not have any adverse effect on the plant safety or operation. This modification did not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.



PLANT CHANGE/MODIFICATION 90-054-01

UNIT : 3/4
TURN OVER DATE : 12/18/91

ENVIRONMENTAL QUALIFICATION OF AIW AND CHAMPLAIN CABLE COMPANY
WIRES

Summary:

This Engineering Package issues Environmental Qualification Documentation Packages for American Insulated Wire Co. #16 AWG 2/c shielded twisted pair cable and Champlain Cable Co. jumper wires. These wires were identified in Safety Injection flow transmitter circuits within the scope of 10CFR50.49.

Safety Evaluation:

This package is classified as Safety-Related since the circuits in which these cables are utilized are safety related circuits. This Engineering Package represents a documentation change only and does not include any plant modifications. Thus, the modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.

PLANT CHANGE/MODIFICATION 90-070-01

UNIT : 3
TURN OVER DATE : 08/17/91

UNIT 3 LOAD CENTER AND SWITCHGEAR ROOMS CHILLED WATER AIR
CONDITIONING SYSTEM

Summary:

This Engineering Package (EP) provides the design, documentation, and installation instructions for the modifications to replace the existing air conditioning (A/C) system with a new upgraded A/C system for the Unit 3 Load Center and Switchgear Rooms (i.e., Fire Zones 070, 071, 095 and 096). In addition, this EP replaces the existing ventilation fan and associated equipment in Load Center Room 3A/3B.

Safety Evaluation:

The replacement of the existing A/C system enhances the capability of providing a temperature controlled environment for the Safety Related equipment located within the Load Center and Switchgear Rooms. This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.

PLANT CHANGE/MODIFICATION 90-071-01

UNIT : 4
TURN OVER DATE : 09/04/91

UNIT 4 LOAD CENTER AND SWITCHGEAR ROOMS CHILLED WATER AIR
CONDITIONING SYSTEM

Summary:

This Engineering Package (EP) provides the design, documentation, and installation instructions for the modifications to replace the existing air conditioning (A/C) system with a new upgraded A/C system for the Unit 4 Load Center and Switchgear Rooms (i.e., Fire Zones 067, 068, 093, and 094). In addition, this EP replaces the existing ventilation fan and associated equipment in Load Center Room 4A/4B.

Safety Evaluation:

The replacement of the existing A/C system enhances the capability of providing a temperature controlled environment for the Safety Related equipment located within the Load Center and Switchgear Rooms. This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.



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.. PLANT CHANGE/MODIFICATION 90-095-01

UNIT : 3
TURN OVER DATE : 07/20/91

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UNIT 3 REPLACEMENT OF VALVE 3-270A

Summary:

This Engineering Package established the design equivalence and conformance to the original design basis for the replacement of valve 3-270A and provides the requirements for its installation.

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.



PLANT CHANGE/MODIFICATION 90-100

UNIT : 3/4
TURN OVER DATE : 12/31/91

REPLACEMENT OF INSTRUMENT TUBING AT THE INTAKE STRUCTURE

Summary:

This Engineering Package provides for the replacement of the existing temporary tubing at the intake structure area. The tubing is associated with the Screen Wash System differential pressure instrumentation for Units 3 and 4, and the ICW pressure indicators (PIs)/pressure switches (PSS) for Unit 3.

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.

PLANT CHANGE/MODIFICATION 90-144

UNIT : 3
TURN OVER DATE : 08/03/91

UNIT 3 PLATFORM FOR LOAD CENTER AND SWITCHGEAR ROOM A/C CHILLERS

Summary:

This Engineering Package provides the necessary design, documentation, and instructions for adding a steel-framed structure to support the new chilled water units and related components associated with the Unit 3 Load Center and Switchgear Room Air Conditioning (A/C) System.

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.

PLANT CHANGE/MODIFICATION 90-146

UNIT : 4
TURN OVER DATE : 08/08/91

UNIT 4 PLATFORM FOR UNIT 4 LOAD CENTER AND SWITCHGEAR ROOM A/C
CHILLERS

Summary:

This Engineering Package provides the necessary design, documentation, and instructions for adding a steel-framed structure to support the new chilled water units and related components associated with the Unit 4 Load Center and Switchgear Room Air Conditioning System.

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.

PLANT CHANGE/MODIFICATION 90-149-01

UNIT : 3/4
TURN OVER DATE : 09/05/91

INSTALLATION OF VITAL AREA BARRIER CRANES

Summary:

This Engineering Package provides the necessary design, documentation and instructions for the installation of 4 base plate weldments. The base plate weldments will be used for installation of mast-mounted pedestal cranes to support construction work activities associated with the installation of the Vital Area Barriers.

Safety Evaluation:

Installation of the temporary pedestal crane supports adjacent to the main steam platforms does not change the design bases, functions or operations of any safety related equipment and does not adversely affect any other safety related structures, systems or components. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.

PLANT CHANGE/MODIFICATION 90-153-02

UNIT : 3
TURN OVER DATE : 08/21/91

UNIT 3 REACTOR PROTECTION SYSTEM DRAWINGS UPGRADE FOR NRC AUDIT
89-203

Summary:

This Engineering Package provides for the necessary revisions to the Reactor Protection System (RPS) design drawings to resolve the errors and deficiencies identified during the NRC Audit 89-203. The revisions to the RPS drawings involved corrections of various type discrepancies (e.g. drafting/administrative wiring mistakes, improperly identified switches, ambiguous terminal point identification, outdated drawing references) and enhancements to provide for improved readability, accuracy, and technical content. In addition, the drawings will be split to have independent drawings for Units 3 and 4 and Trains A and B with 5610-M-430-146, SH, 6 upgraded in accordance with the standard format used for Turkey Point elementary diagrams.

Safety Evaluation:

There are no physical plant modifications associated with this package and hence, no effect on plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.

PLANT CHANGE/MODIFICATION 90-154-02

UNIT : 4
TURN OVER DATE : 08/21/91

UNIT 4 REACTOR PROTECTION SYSTEM DRAWINGS UPGRADE FOR NRC AUDIT
89-203

Summary:

This Engineering Package provides for the necessary revisions to the Reactor Protection System (RPS) design drawings to resolve the errors and deficiencies identified during NRC Audit 89-203. The revisions to the RPS drawings involved corrections of various type discrepancies (e.g. drafting/administrative wiring mistakes, improperly identified switches, ambiguous terminal point identification, outdated drawing references) and enhancements to provide for improved readability, accuracy, and technical content. In addition, the drawings will be split to have independent drawings for Units 3 and 4 and Trains A and B with 5610-M-430-146, SH, 6 upgraded in accordance with the standard format used for Turkey Point elementary diagrams.

Safety Evaluation:

There are no physical plant modifications associated with this package and hence, no effect on plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.

PLANT CHANGE/MODIFICATION 90-155-04

UNIT : 3/4
TURN OVER DATE : 08/06/91

INSTALLATION OF TEMPORARY CABLES FOR TESTING OF
NEW EDGS 4A AND 4B

Summary:

This Engineering Package provides details for the temporary cables, protection and breaker controls designed to facilitate testing of the new EDGs. These modifications shall include disconnecting Cranking Diesel Generator #2 from 4KV switchgear breaker 1W147 and utilizing that breaker to connect two new EDGs to the grid, one at a time for load testing. The load testing of the new EDGs shall be performed prior to permanent connection to the existing emergency power system.

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.



PLANT CHANGE/MODIFICATION 90-176

UNIT : 3/4
TURN OVER DATE : 03/23/92

AUXILIARY BUILDING EXHAUST FAN INTAKE DAMPER MODIFICATION

Summary:

This Engineering Package provides details for the following modifications which have been implemented:

1. Disconnect the electrical power supply to the Exhaust Fan Intake Dampers (MO-3409 and MO-3410) and remove the motor and the linkage between the motor and the damper rod.
2. Remove the limit switches (associated with the intake dampers MO-3409 and MO-3410) and the interlock between the limit switches and the exhaust fans V8A and V8B.
3. Provide capability to keep the exhaust fan intake dampers (MO-3409 and MO-3410) manually "blocked open" in the Auxiliary Building Exhaust Fan Room Enclosure. These dampers will always remain "blocked open" during plant operation.

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.



PLANT CHANGE/MODIFICATION 90-178

UNIT : 4
TURN OVER DATE : 08/14/91

UNIT 4 REMOVAL OF PIPE SUPPORT 795A-10 LOCATED ON THE COMPONENT
COOLING WATER LINE FROM THE "A" RCP THERMAL BARRIER COOLER INSIDE
UNIT 4 CONTAINMENT, STRESS PROBLEM "CCW-42"

Summary:

This Engineering Package provides the documentation necessary to remove pipe support 795A-10 located on the Component Cooling Water line from the "A" RCP thermal barrier cooler located inside Unit 4 Containment. The piping is part of the CCW pressure boundary and is categorized as Quality Group "C", Safety Related. Accordingly, the pipe support removal is considered Safety Related.

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.

PLANT CHANGE/MODIFICATION 90-220-01

UNIT : 3
TURN OVER DATE : 09/08/91

UNIT 3 RTD BYPASS ELIMINATION MODIFICATION AND EAGLE 21
INSTALLATION

Summary:

This Engineering Package (EP) provides the design for removing the existing bypass manifold piping with its direct immersion RTDs and for modifying the manifold connections to the main loop piping to accept thermowells into which dual element fast-response RTDs will be mounted. The EP also provides the design for new penetrations which are required due to existing interferences. In addition, the equalizing supply to the RHR inlet valve (MOV-3-750) will be removed with the RTD bypass piping.

This EP includes the design for removing the existing analog protection modules and circuits used in the T_{avg} , Delta T, and pressurizer level protection functions and replacing them with a Digital System that calculates a T_{hot} average temperature and implements the protection functions presently provided.

This EP provides the design for modifying the non-1E control circuitry which is presently supplied by signals from non-1E RTDs in the manifold.

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question but do require changes to the plant Technical Specifications. Therefore, NRC approval of this modification will be required prior to entry into Mode 6 from the Dual Unit Outage.



PLANT CHANGE/MODIFICATION 90-221-02

UNIT : 4
TURN OVER DATE : 09/30/91

UNIT 4 RTD BYPASS ELIMINATION MODIFICATION AND EAGLE 21
INSTALLATION

Summary:

This Engineering Package (EP) provides the design for removing the existing bypass manifold piping with its direct immersion RTDs and for modifying the manifold connections to the main loop piping to accept thermowells into which dual element fast-response RTDs will be mounted. The EP also provides the design for new penetrations which are required due to existing interferences. In addition, the equalizing supply to the RHR inlet valve (MOV-4-750) will be removed with the RTD bypass piping.

This EP includes the design for removing the existing analog protection modules and circuits used in the T_{avg} , Delta T, and pressurizer level protection functions and replacing them with a Digital System that calculates a T_{hot} average temperature and implements the protection functions presently provided.

This EP provides the design for modifying the non-1E control circuitry which is presently supplied by signals from non-1E RTDs in the manifold.

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question but do require changes to the plant Technical Specifications. Therefore, NRC approval of this modification will be required prior to entry into Mode 6 from the Dual Unit Outage.



PLANT CHANGE/MODIFICATION 90-225-04

UNIT : 3
TURN OVER DATE : 09/09/91

UNIT 3 RTD BYPASS ELIMINATION CABLE INSTALLATION

Summary:

This Engineering Package provides for the installation of allocated raceway and cables required to support the following modifications from the RTDs to the penetrations and from the penetrations to the QR racks:

1. Installation of digital Reactor Protection System cabinets (Eagle-21) for the overpower and overtemperature protection and pressurizer level channels,
2. Replacement of the analog Hagan modules from cabinets 3QR1, 3QR11, and 3QR14 with Eagle structured equipment,
3. Replacement of the existing single element Hot and Cold Leg Resistance Temperature Detectors (RTDs) with dual element RTDs.

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.

PLANT CHANGE/MODIFICATION 90-226-04

UNIT : 4
TURN OVER DATE : 10/17/91

UNIT 4 RTD BYPASS ELIMINATION CABLE INSTALLATION

Summary:

This Engineering Package provides for the installation of allocated raceway and cables required to support the following modifications from the RTDs to the penetrations and from the penetrations to the QR racks:

1. Installation of digital Reactor Protection System cabinets (Eagle-21) for the overpower and overtemperature protection and pressurizer level channels,
2. Replacement of the analog Hagan modules from cabinets 4QR1, 4QR11, and 4QR14 with Eagle structured equipment,
3. Replacement of the existing single element Hot and Cold Leg Resistance Temperature Detectors (RTDs) with dual element RTDs.

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.

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.. PLANT CHANGE/MODIFICATION 90-229-01

UNIT : 4
TURN OVER DATE : 08/06/91

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UNIT 4 SGBD SAFETY VALVE RV-4-6271 DISCHARGE EVALUATION

Summary:

This Engineering Package details the design requirements for the extension of the RV-4-6271 discharge piping into the catch basin and subsequent installation of an enclosure over the catch basin cover to prevent injury to personnel. This piping does not perform a safety function and its structural collapse will not impact any safety related components or systems.

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.



PLANT CHANGE/MODIFICATION 90-238-02

UNIT : 3
TURN OVER DATE : 05/11/92

UNIT 3 C BUS SWITCHGEAR CONTROL AND PROTECTION POWER ISOLATION
FOR APPENDIX R

Summary:

This Engineering Package provides the details for the installation of a molded case circuit breaker at the C Bus switchgear to connect or disconnect the control and protection power for the switchgear. This will eliminate the present fuse pulling requirement and thereby reduce the Operator burden associated with supplying power to the Standby Steam Generator Feedwater Pumps (SSGFPS).

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.

PLANT CHANGE/MODIFICATION 90-249-02

UNIT : 3
TURN OVER DATE : 09/20/91

UNIT 3 ADDITION OF ISOLATION BINS TO THE EXISTING ERDADS
TERMINATION CABINETS

Summary:

This Engineering Package provides for the installation of signal isolators on Safety Parameters Display System (SPDS) non-vital inputs. Input analog isolation will be added to non-isolated SPDS inputs to ensure acceptable non-vital instrument loop accuracy. Digital input isolation will also be provided to improve the reliability of the digital signals, by increasing the signal drive voltage, and to isolate the power supply from potential faults on the digital signals' field cables.

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.



PLANT CHANGE/MODIFICATION 90-250-02

UNIT : 4
TURN OVER DATE : 10/03/91

UNIT 4 ADDITION OF ISOLATION BINS TO THE EXISTING ERDADS
TERMINATION CABINETS

Summary:

This Engineering Package provides for the installation of signal isolators on Safety Parameter Display System (SPDS) non-vital inputs. Input analog isolation will be added to non-isolated SPDS inputs to ensure acceptable non-vital instrument loop accuracy. Digital input isolation will also be provided to improve the reliability of the digital signals, by increasing the signal drive voltage, and to isolate the power supply from potential faults on the digital signals' field cables.

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.

PLANT CHANGE/MODIFICATION 90-289-02

UNIT : 3
TURN OVER DATE : 08/14/91

UNIT 3 REDIRECTION OF EXHAUST FLOW FROM UNIT 3 STEAM GENERATOR
FEED PUMP ROOM EXHAUST FANS

Summary:

This Engineering Package provides for the modification of the the existing Steam Generator Feed Pump (SGFP) Room Exhaust Fan assemblies to redirect the exhaust flow of the fans away from the Load Center and Switchgear Rooms chilled water air conditioning system equipment and the associated support platform.

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.



PLANT CHANGE/MODIFICATION 90-340-01

UNIT : 3/4
TURN OVER DATE : 10/25/91

PLANT LIGHTING UPGRADE VITAL AREA BARRIERS

Summary:

This Engineering Package (EP) adds, replaces and/or supplements the existing normal lighting system within the Vital Area Barriers (VAB) based on a lighting survey. Areas affected by the installation of the VAB include the Units 3 and 4 Main Steam Platforms El. 53'6", Auxiliary Feedwater (AFW) Steam Supply Platforms, El. 39'3" and 42'0", Operating Decks, El. 42'0", Condensate Storage Tanks, Unit 3 AFW Valve Access Platform, El. 28'0", and Unit 4 isolation valves enclosure. This EP also provides for upgrading the lighting levels in selected areas adjacent to the exterior perimeter of the Vital Area Barriers (VAB) for a minimum distance of four (4) feet away from the VAB. The areas include the Steam Generator Wet Layup pumps in Unit 4, Units 3 & 4 Blowdown Tanks, Unit 3 Condensate Storage Tank, Unit 4 Breathing Air Containment Penetration Area on El. 18'0" and the Units 3 & 4 5A Feedwater Heaters located on El. 42'0.

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.



PLANT CHANGE/MODIFICATION 90-385-02

UNIT : 3
TURN OVER DATE : 08/17/91

UNIT 3 - DRILLING OF DISC ON MOV-3-750

Summary:

This Engineering Package provides for the drilling of a small pressure-relieving hole in the disc on the RCS side of MOV-3-750. This modification will provide a means of equalizing the pressure in the bonnet with line pressure preventing a hydraulic lock. A hydraulic lock will prevent this valve from opening when required for RHR operation. This valve is located in the RHR system and provides isolation of the RHR pump suction from the RCS.

Safety Evaluation:

This modification will not affect the operation of the RHR system, the operation of any other plant system, or the safety of the plant. Therefore, this modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.



PLANT CHANGE/MODIFICATION 90-386-02

UNIT : 4
TURN OVER DATE : 08/29/91

UNIT 4 - DRILLING OF DISC ON MOV-4-750

Summary:

This Engineering Package provides for the drilling of a small pressure-relieving hole in the disc on the RCS side of MOV-4-750. This modification will provide a means of equalizing the pressure in the bonnet with line pressure preventing a hydraulic lock. A hydraulic lock will prevent this valve from opening when required for RHR operation. This valve is located in the RHR system and provides isolation of the RHR pump suction from the RCS.

Safety Evaluation:

This modification will not affect the operation of the RHR system, the operation of any other plant system, or the safety of the plant. Therefore, this modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.



PLANT CHANGE/MODIFICATION 90-388

UNIT : 3
TURN OVER DATE : 10/25/91

UNIT 3 SPENT FUEL POOL PUMPS - ISOLATION VALVE INSTALLATION

Summary:

This Engineering Package provides the necessary details for modifications to the Spent Fuel Pool pump suction piping which include the addition of isolation valves and increasing the pipe diameter from 8" to 10".

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.



PLANT CHANGE/MODIFICATION 90-390

UNIT : 3/4
TURN OVER DATE : 06/12/92

PLANT PERIMETER FLOODWALL REPAIR

Summary:

This Engineering Package provides the engineering documentation required for plant perimeter floodwall repair/modification and the preparation of as-built of Plant drawings to reflect the configuration of the plant perimeter floodwall.

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.

PLANT CHANGE/MODIFICATION 90-391-01

UNIT : 3/4
TURN OVER DATE : 09/30/91

FSAR REVISION OF REGULATORY GUIDE 1.97

Summary:

This Engineering Package provides the revision of Section 7.5.4 of the Turkey Point FSAR for Units 3 and 4. This section addresses plant commitments to Regulatory Guide 1.97 (Revision 3). These revisions to the FSAR Section 7.5.4 clarify and enhance Turkey Point's existing commitment to Regulatory Guide 1.97 but do not result in any additional plant modifications.

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.

PLANT CHANGE/MODIFICATION 90-393

UNIT : 3/4
TURN OVER DATE : 09/04/91

MODIFICATION OF SUPPORTS 80117-H-331-04, 80117-H-332-03,
AND 80117-H-332-01

Summary:

This Engineering Package provides details to correct the discrepancies documented in NCR N-91-0607. As part of the As-built Verification effort it has been determined that the existing supports Mark No. 80117-H-331-04, 80117-H-332-03, and 80117-H-332-01 do not conform to the original design. These three supports provide support to the Auxiliary Steam System piping and are located on the auxiliary steam platform of the turbine deck.

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.



PLANT CHANGE/MODIFICATION 90-404

UNIT : 4
TURN OVER DATE : 10/20/91

UNIT 4 REACTOR COOLANT PUMP MOTOR MODIFICATIONS AND UPGRADES

Summary:

This Engineering Package provides details for the following Unit 4 reactor coolant pump motor modifications and upgrades:

- Oil Lift System modification
- Lower Cooling Coil modification
- Thrust Runner Seal modification
- Spare Resistance Temperature Detector modification
- Lower Bearing Seal upgrade
- Oil Pan Upgrade
- Upper Bracket Standpipe Modification

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.



PLANT CHANGE/MODIFICATION 90-405

UNIT : 3
TURN OVER DATE : 08/14/91

RECONFIGURE 3 FEEDWATER ISOLATION RESET
OT-2 PUSH BUTTON SWITCHES FOR UNIT 3

Summary:

This Engineering Package (EP) provides the details to reconfigure three OT-2 push button switches such that train A and train B are located on opposite sides of the contact blocks. The push buttons are used in the Feedwater Isolation portion of the Safeguards circuitry, therefore, this EP is considered Safety Related.

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.



PLANT CHANGE/MODIFICATION 90-406

UNIT : 4
TURN OVER DATE : 08/14/91

RECONFIGURE 3 FEEDWATER ISOLATION RESET
OT-2 PUSH BUTTON SWITCHES FOR UNIT 4

Summary:

This Engineering Package (EP) provides the details to reconfigure three OT-2 push button switches such that train A and train B are located on opposite sides of the contact blocks. The push buttons are used in the Feedwater Isolation portion of the Safeguards circuitry, therefore, this EP is considered Safety Related.

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.

PLANT CHANGE/MODIFICATION 90-417-01

UNIT : 4
TURN OVER DATE : 09/12/91

UNIT 4 ABANDONED PRESSURIZER SPRAY VALVE BODY WELDED PLATE

Summary:

This Engineering Package details a modification which provides additional assurance against leakage from the abandoned pressurizer spray valve body to bonnet joints by means of installing a welded plate inside the valve body which is sealed with a blind flange. Because the original gasketed bonnet to body joint is retained as the primary pressure boundary against gross RCS leakage, the welded plate is defined as a modification which improves equipment service.

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.



PLANT CHANGE/MODIFICATION 90-423-01

UNIT : 3
TURN OVER DATE : 08/20/91

UNIT 3 BORIC ACID TRANSFER PUMP SEAL REPLACEMENT

Summary:

This engineering package (EP) provides for replacement of the existing double mechanical seal in the Boric Acid Transfer Pumps with single mechanical cartridge seals which do not require an external seal water flushing system. This EP also provides for installation of a seal flushing line between the new seal assembly and the pump discharge as well as removal of the pump seal water supply system including nitrogen supply, seal water tank, seal water supply and return tubing and miscellaneous instrumentation, valves and components.

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.



PLANT CHANGE/MODIFICATION 90-424-01

UNIT : 4
TURN OVER DATE : 08/20/91

UNIT 4 BORIC ACID TRANSFER PUMP SEAL REPLACEMENT

Summary:

This engineering package (EP) provides for replacement of the existing double mechanical seal in the Boric Acid Transfer Pumps with single mechanical cartridge seals which do not require an external seal water flushing system. This EP also provides for installation of a seal flushing line between the new seal assembly and the pump discharge as well as removal of the pump seal water supply system including nitrogen supply, seal water tank, seal water supply and return tubing and miscellaneous instrumentation, valves and components.

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.

PLANT CHANGE/MODIFICATION 90-425-01

UNIT : 4
TURN OVER DATE : 07/05/91

UNIT 4 LIMITORQUE UPGRADES FOR MOV-4-865 A, B, AND C
AND MOV-4-744 A AND B

Summary:

This Engineering Package (EP) defines the requirements for component upgrades and installation of main gearbox relief valves and motor housing T drains in lieu of existing plugs to MOV-4-865 A, B & C. This EP also defines the requirements for the replacement of nylon-insulated wire connectors inside the Dings brake housing of Limitorque motor operated valve operators associated with these valves and MOV-4-744 A & B.

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.



PLANT CHANGE/MODIFICATION 90-433

UNIT : 4
TURN OVER DATE : 10/14/91

UNIT 4 DRILLING OF VALVE WEDGE FOR MOV-4-872

Summary:

This Engineering Package (EP) provides for the drilling of a small pressure relieving hole in the valve wedge on the Reactor Coolant System (RCS) side of MOV-4-872. As documented in INPO SOER 84-7, system pressure in the valve bonnet area may become trapped causing a high differential pressure across the valve disc/wedge and resultant binding during valve opening. This modification will eliminate the potential for such binding. This EP is classified as Safety Related because MOV-4-872 performs Safety Related functions.

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.

PLANT CHANGE/MODIFICATION 90-441-01

UNIT : 4
TURN OVER DATE : 09/05/91

UNIT 4 BORIC ACID CONCENTRATION REDUCTION

Summary:

This Engineering Package (EP) provides the details to perform the modifications necessary to support a reduction in the concentration of boric acid in the boric acid makeup system.

Currently, the concentration is approximately 12% by weight which requires certain provisions to keep the temperature of the solution in the storage tanks and piping high enough to ensure solubility. This minimum temperature requirement for the solution is higher than the normal ambient temperature in the affected area (Auxiliary Building). Problems such as heat tracing failure, plugged components due to crystalline deposits, caking and corrosion have become commonplace and expensive to deal with.

Safety Evaluation:

The modifications provided by this EP do not have an adverse effect on plant safety or security. However, the modifications provided by this EP are based on the licensing amendment changes which were submitted to the Nuclear Regulatory Commission (NRC) for approval. Therefore, changes made by the EP are based on the assumption that prior NRC approval for boric acid concentration reduction has been granted.

PLANT CHANGE/MODIFICATION 90-460

UNIT : 4
TURN OVER DATE : 08/07/91

UNIT 4 REPAIR AND MODIFICATION OF INTAKE STRUCTURE

Summary:

This Engineering Package (EP) provides the necessary design, documentation, references and instructions to repair and modify degraded concrete slabs located in the Unit 4 Intake Structure CWP 4A1, CWP 4A2 and CWP 4B2 bays corresponding to Screen Wash, ICW 4A, and ICW 4C pumps, respectively. This EP provides the modifications required to restore the concrete beams supporting the two remaining Unit 4 ICW pumps and the Screen Wash Pump to a condition that meets the original design intent of the slabs.

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.



PLANT CHANGE/MODIFICATION 90-471

UNIT : 3
TURN OVER DATE : 03/31/92

UNIT 3 COMPONENT COOLING WATER SURGE TANK RIGIDITY UPGRADE

Summary:

This Engineering Package provides the design and documentation for the modifications necessary to adequately stiffen the CCW surge tank support structure such that the original assumptions of fully rigid anchor points at tank nozzles are valid.

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.



PLANT CHANGE/MODIFICATION 90-472

UNIT : 4
TURN OVER DATE : 07/18/91

UNIT 4 COMPONENT COOLING WATER SURGE TANK RIGIDITY UPGRADE

Summary:

This Engineering Package provides the design and documentation for the modifications necessary to adequately stiffen the CCW surge tank support structure such that the original assumptions of fully rigid anchor points at tank nozzles are valid.

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.



PLANT CHANGE/MODIFICATION 90-482

UNIT : 3
TURN OVER DATE : 08/15/91

UNIT 3 - RECONFIGURE THE CONTAINMENT SPRAY RESET OT-2
PUSH BUTTON SWITCH

Summary:

This Engineering Package (EP) provides the details necessary to reconfigure an OT-2 push button switch such that train A and train B are located on opposite sides of the contact blocks. The push button is used in the Containment Spray portion of the Safeguards circuitry. Therefore, this EP is considered Safety Related.

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.



PLANT CHANGE/MODIFICATION 90-483

UNIT : 4
TURN OVER DATE : 08/15/91

UNIT 4 RECONFIGURE THE CONTAINMENT SPRAY RESET OT-2
PUSH BUTTON SWITCH

Summary:

This Engineering Package (EP) provides the details necessary to reconfigure an OT-2 push button switch such that train A and train B are located on opposite sides of the contact blocks. The push button is used in the Containment Spray portion of the Safeguards circuitry. Therefore, this EP is considered Safety Related.

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.



PLANT CHANGE/MODIFICATION 90-493-02

UNIT : 3/4
TURN OVER DATE : 07/20/91

PUBLIC ADDRESS SYSTEM SOUTHERN PLANT AREA

Summary:

This Engineering Package provides the design for extending the PA System in the Southern Plant areas to provide paging capability for emergency evacuation alarms and instructions. The upgrade is limited to the Construction Craft Building, Construction Building (Backfit Overflow), Construction Warehouse and along the protected area fence from the south side of the Nuclear Administration Building (NAB) to an area southeast of the Dry Storage Warehouse.

Safety Evaluation:

Credit is not taken for the PA System operation to support operator actions for safe shutdown or accident mitigation, nor to prevent radiological exposure to Plant personnel or the general public. Therefore, this modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.

PLANT CHANGE/MODIFICATION 90-499

UNIT : 3
TURN OVER DATE : 08/17/91

UNIT 3 ABANDONED PRESSURIZER SPRAY VALVE BODY WELDED PLATE

Summary:

This Engineering Package provides details for the modification which provides additional assurance against leakage from the abandoned pressurizer spray valve body to bonnet joints by means of installing a welded plate inside the valve body which is sealed with a blind flange. Because the original gasketed bonnet to body joint is retained as the primary pressure boundary against gross RCS leakage, the welded plate is defined as a modification which improves equipment service.

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.

PLANT CHANGE/MODIFICATION 90-500

Page 1 of 2

UNIT : 3
TURN OVER DATE : 09/09/91

UNIT 3 MOV LIMIT SWITCH ENHANCEMENTS - GENERIC LETTER 89-10

Summary:

This Engineering Package provides the following for the Motor Operated Valves (MOV's) which are included in the package scope:

- a) MOV switch setting policy for MOV limit switches consistent with that developed for IE Bulletin 85-03; including increased torque bypass switch settings to improve operability.
- b) Revised MOV limit switch contact assignments and associated wiring changes to ensure that Control Room valve position indication is maintained following the revision of the torque bypass switch settings included in (a) above.
- c) Relocation of cable terminations from torque switches to limit switches.

The following MOV's are within the scope of the Engineering Package:

<u>MOV #</u>	<u>FUNCTION</u>
3-115C	Volume Control Tank to Charging Pump Suction
3-350	Boric Acid Supply to Charging Pump
3-381	RCP Seal Water Return Line
3-626	Thermal Barrier CCW Return Isolation
3-716A	RCP CCW Supply Isolation
3-716B	RCP CCW Supply Isolation
3-730	RCP Oil Cooler CCW Return Isolation
3-749A	RHR Heat Exchanger "A" CCW Isolation
3-749B	RHR Heat Exchanger "B" CCU Isolation
3-880A	Cont Spray Pump "A" Discharge Isolation
3-880B	Cont Spray Pump "B" Discharge Isolation
3-1417	Containment Cooling Discharge
3-1418	Containment Cooling Return



PLANT CHANGE/MODIFICATION 90-500

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UNIT 3 MOV LIMIT SWITCH ENHANCEMENTS - GENERIC LETTER 89-10

These MOV's are-classified as Safety-Related and are required for safe plant shutdown, maintaining the reactor in a safely shutdown condition and/or preventing or mitigating the consequences of accidents with potential offsite exposures approaching 10 CFR 100 levels.

Safety Evaluation:

This modification did not have any adverse effect on the plant safety or operation. This modification did not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.



PLANT CHANGE/MODIFICATION 90-501

Page 1 of 2

UNIT : 4
TURN OVER DATE : 09/30/91

UNIT 4 MOV LIMIT AND TORQUE SWITCH ENHANCEMENTS -
GENERIC LETTER 89-10

Summary:

This Engineering Package provides the following for the Motor Operated Valves (MOV's) which are included in the package scope:

- a) MOV switch setting policy for MOV limit switches consistent with that developed for IE Bulletin 85-03; including increased torque bypass switch settings to improve operability.
- b) Revised MOV limit switch contact assignments and associated wiring changes to ensure that Control Room valve position indication is maintained following the revision of the torque bypass switch settings included in (a) above.
- c) Relocation of cable terminations from torque switches to limit switches.

The following MOV's are within the scope of the Engineering Package:

<u>MOV#</u>	<u>FUNCTION</u>
4-115C	Volume Control Tank to Charging Pump Suction
4-350	Boric Acid Supply to Charging Pump
4-381	RCP Seal Water Return Line
4-626	Thermal Barrier CCW Return Isolation
4-716A	RCP CCW Supply Isolation
4-716B	RCP CCW Supply Isolation
4-730	RCP Oil Cooler CCW Return Isolation
4-749A	RHR Heat Exchanger "A" CCW Isolation
4-749B	RHR Heat Exchanger "B" CCW Isolation
4-880A	Cont Spray Pump "B" Discharge Isolation
4-880B	Cont Spray Pump "A" Discharge Isolation
4-1417	Containment Cooling Discharge
4-1418	Containment Cooling Return

PLANT CHANGE/MODIFICATION 90-501

Page 2 of 2

UNIT 4 MOV LIMIT AND TORQUE SWITCH ENHANCEMENTS -
GENERIC LETTER 89-10

These MOV's are classified as Safety-Related and are required for safe plant shutdown, maintaining the reactor in a safely shutdown condition and/or preventing or mitigating the consequences of accidents with potential offsite exposures approaching 10 CFR 100 levels.

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.

PLANT CHANGE/MODIFICATION 90-518-04

UNIT : 3
TURN OVER DATE : 08/19/91

UNIT 3 REPLACEMENT OF MOV 535/536 LIMITORQUE OPERATORS

Summary:

The Engineering Package provides for the replacement of the Unit 3 PORV Block Valves' (MOV-3-535,536) operators. The replacement operators will provide increased reliability due to superior torque switch and compartment design as well as increased margin between the maximum torque available and the torque required to overcome differential pressure. The replacement of the operators as well as minor changes required by that replacement (new valve wedge material, replacement of thermal overload heaters, installation of compatible valve yoke) have been demonstrated to be equivalent to the existing configuration in regard to form, fit, and function. Therefore, this replacement does not constitute a change to any design bases.

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.



PLANT CHANGE/MODIFICATION 90-519-04

UNIT : 4
TURN OVER DATE : 10/25/91

UNIT 4 REPLACEMENT OF MOV 535/536 LIMITORQUE OPERATORS

Summary:

The Engineering Package provides for the replacement of the Unit 4 PORV Block Valves' (MOV-4-535,536) operators. The replacement operators will provide increased reliability due to superior torque switch and compartment design as well as increased margin between the maximum torque available and the torque required to overcome differential pressure. The replacement of the operators as well as minor changes required by that replacement (new valve wedge material, replacement of thermal overload heaters, installation of compatible valve yoke) have been demonstrated to be equivalent to the existing configuration in regard to form, fit, and function. Therefore, this replacement does not constitute a change to any design bases.

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.

PLANT CHANGE/MODIFICATION 90-528

UNIT : 3
TURN OVER DATE : 09/09/91

UNIT 3 PRESSURIZER PRESSURE TRANSMITTER REPLACEMENT

Summary:

This Engineering Package (EP) provides for the replacement of existing Rosemount transmitters used in the Pressurizer Pressure - Low pressure Safety Injection (SI) Reactor Protection System actuation function, with Rosemount Model 1154 Series H transmitters. This EP provides for the change of span of the Rosemount 1154 Series H protection transmitters to a span of 1500 to 2500 psig, and an associated reduction in the safety analysis limit for low pressurizer pressure SI actuation from the current value of 1700 psig to 1600 psig. In addition to replacing and changing the span of the protection transmitters, this EP will also require the change of span of the control transmitters to maintain overall system compatibility.

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.



PLANT CHANGE/MODIFICATION 90-529

UNIT : 4
TURN OVER DATE : 09/17/91

UNIT 4 PRESSURIZER PRESSURE TRANSMITTER REPLACEMENT

Summary:

This Engineering Package (EP) provides for the replacement of existing Rosemount transmitters used in the Pressurizer Pressure - Low pressure Safety Injection (SI) Reactor Protection System actuation function, with Rosemount Model 1154 Series H transmitters. This EP provides for the change of span of the Rosemount 1154 Series H protection transmitters to a span of 1500 to 2500 psig, and an associated reduction in the safety analysis limit for low pressurizer pressure SI actuation from the current value of 1700 psig to 1600 psig. In addition to replacing and changing the span of the protection transmitters, this EP will also require the change of span of the control transmitters to maintain overall system compatibility.

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.

PLANT CHANGE/MODIFICATION 90-531

UNIT : 3
TURN OVER DATE : 03/04/92

UNIT 3 REPLACEMENT OF RCP MOTOR 3A WITH THE SPARE MOTOR

Summary:

This Engineering Package documents the installed condition for the replacement of the Reactor Coolant Pump (RCP) 3A with a spare motor. The spare motor was purchased by FPL Maintenance and installed to replace RCP Motor 3A in 1982.

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.



PLANT CHANGE/MODIFICATION 90-535-02

UNIT : 3
TURN OVER DATE : 09/11/91

UNIT 3 GENERATOR ROTOR RETAINING RING MODIFICATION

Summary:

This Engineering Package addresses the material changes that are being performed in association with the generator rotor rewind. The generator is not associated with safe plant shutdown or removal of post-accident heat loads.

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.



PLANT CHANGE/MODIFICATION 90-539

UNIT : 3/4
TURN OVER DATE : 09/18/91

MODIFICATION OF THE SPENT FUEL ASSEMBLY HANDLING TOOL

Summary:

This Engineering Package provides the necessary engineering required for qualification of the new spent fuel assembly handling tools, incorporation of the drawing into the Document Control System, and appropriate modification of the tools for proper handling of the spent fuel assemblies.

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.

PLANT CHANGE/MODIFICATION 90-543

UNIT : 3/4
TURN OVER DATE : 09/07/91

SETPOINT CHANGE FOR PCV-*-6180

Summary:

This Engineering Package provides for justification necessary to reset PCV-*-6180 from 15 psig to 37.5 psig to insure sufficient air flow to the breathing air bubble hoods. The Breathing Air System is not safety related and, except for the piping and valves at the containment penetration, serves no safety function.

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.

PLANT CHANGE/MODIFICATION 91-004

UNIT : 3
TURN OVER DATE : 07/20/91

UNIT 3 MOV ENHANCEMENT - GENERIC LETTER 89-10

Summary:

This Engineering Package provides details for enhancements to the motor operators of selected safety related motor operated valves required to ensure the valves will function during maximum delta P conditions at minimum voltage availability.

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.

PLANT CHANGE/MODIFICATION 91-005

UNIT : 4
TURN OVER DATE : 07/20/91

UNIT 4 MOV ENHANCEMENT - GENERIC LETTER 89-10

Summary:

This Engineering Package provides details for enhancements to the motor operators of selected safety related motor operated valves required to ensure the valves will function during maximum delta P conditions at minimum voltage availability.

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.



PLANT CHANGE/MODIFICATION 91-026

UNIT : 3/4
TURN OVER DATE : 08/20/91

REMOVAL OF ACID TANK EQUALIZING LINE

Summary:

This Engineering Package (EP) evaluates the abandonment and removal of the acid tank equalizing line which was performed as a temporary repair by NCR N-90-0747. Neither the acid tank equalizing line nor the acid tanks are shown on the ISI Code Boundary Drawings. The acid tanks are not required for a safe shutdown of the plant, nor do they mitigate the consequences of an accident. This EP is an administrative document change only, and has been prepared to update the Plant drawings, procedures and data bases.

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.

PLANT CHANGE/MODIFICATION 91-028

UNIT : 3/4
TURN OVER DATE : 09/19/91

PROVISIONS FOR THE INSTALLATION AND OPERATION OF A POST-ACCIDENT
HYDROGEN RECOMBINER

Summary:

This Engineering Package provides details for the facility and procedural changes required to accommodate the installation and operation of a 90 SCFM (nominal) Hydrogen Recombiner instead of the original 70 SCFM (nominal) Hydrogen Recombiner. The Hydrogen Recombiner is provided as an alternative to venting the containment atmosphere through the Post Accident Containment Ventilation System (PACVS) to control post-accident hydrogen concentrations.

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.

PLANT CHANGE/MODIFICATION 91-035-01

UNIT : 4
TURN OVER DATE : 07/03/91

UNIT 4 STEAM GENERATOR TUBE PLUGGING

Summary:

This Engineering Package provides for the installation of mechanical tube plugs in the Turkey Point Unit 4 steam generators. Tubes may be plugged if previously installed plugs are removed, or if in-service (eddy-current) inspection indicates plugging is required. The plugging is not to exceed the Technical Specification limit of 160 plugs total per steam generator.

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.

UNIT : 4
TURN OVER DATE : 10/31/91

UNIT 4 REMOVAL OF STARTUP SOURCES

Summary:

This engineering package provides the necessary documentation of the technical criteria, operational and maintenance guidelines, and document revision information required for removal of the startup sources at Turkey Point Unit 4. Startup neutron sources have been used at both Turkey Point Units to increase the count rate at the excore source range neutron detectors during refueling operations and during the initial approach to criticality to enhance the capability to monitor subcritical multiplication in the core.

The startup sources are used to enhance the count rate on the source range detectors which measure the neutron count rate (i.e., subcritical multiplication) during refueling and during the initial approach to criticality. They are used to provide an audible indication of the neutron count rate in containment to the control room and to the operators in containment to prevent high personnel exposure during refueling operations. The detectors also provide a reactor trip signal in the event of an excessively high startup rate.

Calculation PTN-BFJF-91-01, "Neutron Source Requirements During Reload and Initial Criticality - Turkey Point Units 3 & 4," demonstrated that the residual neutron production from fission product decay in the reloaded fuel alone provides a detector response sufficiently above background to monitor subcritical multiplication using the source range detectors. The analysis considered a broad range of fuel initial enrichments, burnups, and decay times. Therefore, the refueling procedures for each unit will be revised to place fuel assemblies with sufficient neutron sources closest to the excore source range detectors to assure an adequate detector response during refueling operations. The FSAR will be updated to reflect that the reloaded fuel will provide an adequate source of neutrons for future cycles.

PLANT CHANGE/MODIFICATION 91-043

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UNIT 4 REMOVAL OF STARTUP SOURCES

Safety Evaluation:

This safety evaluation shows that removal of the startup sources does not have any adverse effect on plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.

PLANT CHANGE/MODIFICATION 91-059

UNIT : 3
TURN OVER DATE : 07/15/91

UNIT 3 HIGH PRESSURE TURBINE HORIZONTAL JOINT SEALING GROOVE

Summary:

During the disassembly of the Unit 3 High Pressure Turbine during the 1990-1991 Dual Unit Outage, it was discovered that a sealing pumping groove in the cylinder cover horizontal joint already existed. This Engineering Package (EP) documents the existence of the sealing pumping groove, the relative injection port locations, and the groove dam locations which enable a localized area of the joint to be injected and sealed. Additionally, this EP documents the steam erosion repair of the cylinder cover horizontal joint east side gland areas (both governor and generator ends).

Safety Evaluation:

This EP documents the As-Built Configuration of the Unit 3 High Pressure Turbine horizontal joint and shows that the configuration of the horizontal joint meets the intended design requirements. This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.



PLANT CHANGE/MODIFICATION 91-087

UNIT : 4
TURN OVER DATE : 08/27/91

UNIT 4 REPLACING VALVE 4-519 STOP CHECK WITH CHECK VALVE

Summary:

This Engineering Package provides the necessary modifications to the Nitrogen System such that the existing 3/4" Stainless Steel (S.S.) Stop Check Valve (at Valve Tag Number 4-519) may be replaced with a 3/4" Carbon Steel (C.S.) Piston Check Valve.

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.



PLANT CHANGE/MODIFICATION 91-092

UNIT : 3/4
TURN OVER DATE : 10/01/91

REMOVAL OF BORIC ACID TANK HEADER

Summary:

This Engineering Package provides for the removal and capping of a section of Chemical and Volume Control System (CVCS) piping that is currently installed in the Boric Acid Tank Room. Specifically, the pipe section to be removed is the header that connects the three Boric Acid Tanks (BATs) to the Concentrates Holdup Tank (CHT) transfer pumps and the Boric Acid Recirculation Pumps.

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.



PLANT CHANGE/MODIFICATION 91-134

UNIT : 3/4
TURN OVER DATE : 02/19/92

MAINSTREAM PLATFORM MONORAIL TROLLEY UPGRADE

Summary:

This Engineering Package (EP) involves an administrative change only with no plant modifications required. The primary purpose of this EP is to increase the allowable load capacity of selected Unit 3 & 4 mainstream platform monorail trolleys from 1500 lbs to 2500 lbs. This package also established requirements associated with the use of all of the Unit 3 & 4 mainstream platform monorail trolleys purchased under CRN's 5-5236 and C-5237.

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.

PLANT CHANGE/MODIFICATION 91-141

UNIT : 3
TURN OVER DATE : 10/01/91

UNIT 3 MODIFICATIONS OF MAIN STEAM SYSTEM PRESSURE TRANSMITTER
PT-3-484 INSTRUMENT TUBING

Summary:

This Engineering Package provides for the closing of valve 3-10-202 and the re-routing of the sensing line for PT-3-484. The sensing line for PT-3-484 will be routed to the tubing associated with valve 3-10-251 (isolation valve for PT-3-1607). Additionally, plant procedures and drawings will be updated to reflect the closure of valve 3-10-202 and the revised tubing configuration.

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.

PLANT CHANGE/MODIFICATION 91-143

UNIT : 3/4
TURN OVER DATE : 02/18/92

WEATHER IMPROVEMENT FOR CASK WASH AREA RADIATION MONITORING
CHANNELS RD-4-1411 AND RD-3-1412

Summary:

The Victoreen Area Radiation Monitoring System (ARMS) channels RD4-1411 and RD-3-1412 detect, indicate and provide alarm for the cask wash area in Units 3 and 4. The detector, preamplifier box and local meter display and alarm box are installed on the wall of the cask wash area which is located in an open area exposed to rain and sunlight. Under heavy weather conditions, the amplifier boxes and the detectors have been subjected to water intrusion causing false upscale indication in the Control Room. This Engineering Package provides for the design and instructions to enclose the preamplifier box into a water resistant box and invert the position of the detector to prevent water intrusions and the corresponding false readings.

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.



PLANT CHANGE/MODIFICATION 91-146

UNIT : 3/4
TURN OVER DATE : 12/03/91

REPLACEMENT OF TURBINE SUPERVISORY LOCAL TACHOMETER

Summary:

This Engineering Package (EP) evaluates the replacement of the existing local tachometers (one per unit) with 37506A-01 Digital Tachometers. The local tachometers are located on the main turbine deck at the governor pedestal in panels 3C100 and 4C100. This EP demonstrates the acceptability of the replacement tachometers for the main turbine.

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.

PLANT CHANGE/MODIFICATION 91-147

UNIT : 4
TURN OVER DATE : 11/08/91

UNIT 4 INTERCEPT CONTROL VALVE GUARDED OIL PIPING SUPPORT
MODIFICATION

Summary:

The Unit 4 North East Intercept valve guarded oil pipe has a history of excessive vibration. This Engineering Package provides for the installation of additional internal supports for the control oil lines inside the guarded oil pipe system, and provides external lateral support of the guarded oil pipe to help minimize the vibration of the overall guarded oil pipe system. These modifications will minimize the possibility of future failure of the threaded and welded joints of the high pressure and control oil piping, minimize the fretting that has occurred on the instrumentation wiring, and minimize the lateral vibration of the guarded oil pipe.

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.



PLANT CHANGE/MODIFICATION 91-150

UNIT : 4
TURN OVER DATE : 12/06/91

UNIT 4 TRANSFORMER COOLING FAN MOUNTING REINFORCING

Summary:

Several mounting brackets supporting the fan motors at the cooler banks on the Unit 4 Main Power Transformer have experienced recent failures. This Engineering Package provides the specifications for the reinforcement of all remaining fan motor supports to prevent future failures.

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.



PLANT CHANGE/MODIFICATION 91-157-01

UNIT : 3
TURN OVER DATE : 01/17/92

UNIT 3 INTERCEPT CONTROL VALVE GUARDED OIL PIPING SUPPORT
MODIFICATION

Summary:

This Engineering Package provides for the installation of additional internal supports for the Unit 3 control oil lines inside the guarded oil pipe system, and provides external lateral support of the Unit 3 guarded oil pipe to help minimize the vibration of the overall guarded oil pipe system. These modifications will minimize the possibility of future failure of the threaded and welded joints of the high pressure and control oil piping, minimize the fretting that has occurred on the instrumentation wiring, and minimize the lateral vibration of the guarded oil pipe.

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.



PLANT CHANGE/MODIFICATION 91-204

UNIT : 3
TURN OVER DATE : 05/05/92

UNIT 3 RESTORATION OF MAIN STEAM SYSTEM PRESSURE TRANSMITTER (PT-3-484) INSTRUMENT TUBING

Summary:

This Engineering Package provides the details to return the existing PT-3-484 tubing configuration to the original design configuration, following replacement of valve 3-10-202. Additionally, plant procedures and drawings will be updated to reflect the re-opening of valve 3-10-202 with hand-wheel installed and the revised tubing configuration.

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.

PLANT CHANGE/MODIFICATION 92-001

UNIT : 3/4
TURN OVER DATE : 03/06/92

REMOVAL OF THE SPENT RESIN HANDLING SYSTEM

Summary:

This Engineering Package removes the de-watering equipment portion of the "permanently installed" Spent Resin Handling Subsystem which has been abandoned. This modification requires the removal of equipment, supports, cables, and conduit. The removal of equipment and pipes associated with this particular sub-system to the Condensate Polishing System will not impose any additional loads on structures or remaining equipment and pipe supports.

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.



PLANT CHANGE/MODIFICATION 92-012

UNIT : 3/4
TURN OVER DATE : 04/01/92

VENDOR MANUAL REVISION TO EXTEND RUNTIME ON ECP MOTORS

Summary:

This Engineering Package (EP) revises Vendor Manual V000060, "Information, Operation and Maintenance Instructions - Emergency Containment Filter Equipment and Cooling Units; Turkey Point Units 3 and 4", pages 14 and 37 in reference to preventative maintenance run time and lubrication. This EP will revise this manual to align the general maintenance information with the information in the instruction manuals. Therefore, this EP is considered administrative only.

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.



PLANT CHANGE/MODIFICATION 92-014-02

UNIT : 3
TURN OVER DATE : 06/25/92

UNIT 3 CATHODIC PROTECTION FOR CCW HEAT EXCHANGERS

Summary:

This Engineering Package provides details for the installation of sacrificial anodes in the Component Cooling Water (CCW) heat exchanger inlet and outlet channel heads in order to return the Intake Cooling Water (ICW) chemical injection system to service. The sacrificial anodes shall provide cathodic protection to minimize future stress corrosion cracking and erosion/corrosion damage.

Safety Evaluation:

This modification does not have any adverse effect on the plant safety or operation. This modification does not constitute an unreviewed safety question or require changes to the plant Technical Specifications. Therefore, prior NRC approval was not required for this modification.

SAFETY EVALUATION: JPE-LR-87-020 Revision 5

OPERATION WITH LOSS OF HVAC TO THE DC EQUIPMENT AND INVERTER ROOMS

In 1986, Florida Power and Light (FPL) identified a concern related to the loss of HVAC in the DC Equipment/Inverter Rooms at Turkey Point Units 3 & 4.

This evaluation indicates a maximum operability temperature of 135 degrees Fahrenheit is acceptable (except for the battery rooms where 115 degrees Fahrenheit is the acceptable maximum). Operation at or above this temperature for short periods of time (from a few days to months depending on the particular piece of equipment) does not introduce a failure mechanism. This evaluation also reviewed the use of supplemental cooling and provided requirements that should be implemented to ensure detection and timely compensatory actions for any credible HVAC failure scenario and resulting temperature excursion in the DC Equipment/Inverter rooms.

Safety Evaluation Summary:

The DC Equipment/Inverter Rooms' HVAC systems are not addressed in the Turkey Point Technical Specifications; therefore, this evaluation has no effect on the plant's Technical Specifications. Since no existing safety analyses are affected and no new failure modes are introduced, the use of supplemental cooling does not constitute an unreviewed safety question pursuant to 10 CFR 50.59. Therefore, prior NRC approval was not required.

Revision 5 of this SE provides additional evaluations which justify a 60 minute delay for manually restarting certain HVAC loads after a Loss of Offsite Power (LOOP) event. Also, air flow rates used for the HVAC systems of the DC Equipment and Inverter Rooms have been updated based on post modification testing performed during the dual unit outage of 1990 - 1991.

Issued: November 6, 1991

SAFETY EVALUATION: JPN-PTN-SEMS-90-041 Revision 4

ACCEPTABILITY OF AS-FOUND CONDITION FOR RHR VALVE 3-753A

The unit 3 Residual Heat Removal System (RHR) "A" Pump Discharge Check Valve 3-753A has been determined to have a 1 1/32" long by 1/16" wide linear indication the full distance across the valve seat and extending into the valve body. A flaw evaluation has been performed in accordance with general flaw evaluation methods contained in ASME Section XI.

Safety Evaluation Summary:

Revision 0 to this evaluation showed that the continued use and operation of check valve 3-753A, in its as-found condition between March 1990, and the Dual Unit Outage did not affect any Technical Specifications, plant operation or safety, and that an unreviewed safety question did not exist.

Revision 1 to this evaluation showed that, based on present flaw condition, continued use and operation of check valve 3-753A is acceptable for the remainder of Unit 3 Cycle 12 (not including cycle 13 fuel load activities).

Revision 2 to this evaluation included administrative changes, clarification to allowed operational durations and miscellaneous consistency changes.

Revision 3 to this evaluation showed that continuous operation without reinspection of 3-753A is acceptable based on the calculated flaw growth to be small for more than 40 years. This effectively extends the acceptable operation period of the valve to the end-of-service life of the plant.

Revision 4 removes the requirement for the re-inspection of check valve 3-753A at the first opportunity after January 1996, originally in Revision 3. Continued operation of valve 3-753A does not represent an operability concern, does not affect any Technical Specifications, plant operation or plant safety, and does not constitute an unreviewed safety question.

Issued: June 26, 1992

SAFETY EVALUATION: JPN-PTN-SEMS-90-066 Revision 0

REMOVAL OF TURBINE OVERSPEED TRIP LOWER RANGE VALUE FROM THE UFSAR

Section 14.1.13 of the UFSAR and plant procedures require that the Mechanical Turbine Overspeed Trip stop all flow of steam into or through the turbine when the turbine speed has increased to the range of 1970 RPM to 1998 RPM (this corresponds to approximately 9 1/2% to 11% above normal speed). This Safety Evaluation addresses the elimination of the listed range of 1970 RPM from the UFSAR because the range is restrictive and can cause unnecessary plant delays during startup. The Safety Evaluation also provides the acceptable range of the trip values that should be maintained.

Safety Evaluation Summary:

The information presented in this safety evaluation demonstrates that the operational function of the Mechanical Turbine Overspeed Trip mechanism upper limit is the critical value and the elimination of the lower limit and the newly defined range as identified in the Safety Evaluation will not adversely effect plant safety or operations. There are no new restrictions on plant operations pursuant to this evaluation.

Based on this evaluation, this change does not constitute an unreviewed safety question or a change to the plant Technical Specification pursuant to 10CFR50.59.

Issued: September 27, 1991



SAFETY EVALUATION: JPN-PTN-SEMJ-90-105 Revision 0

GENERIC LETTER 88-17: DEFEAT OF RHR SUCTION VALVES INTERLOCK DURING MID-LOOP OPERATIONS

This safety evaluation addresses a recommendation in GL 88-17 to implement/revise procedures and administrative controls. These procedures and administrative controls are to avoid operations that can lead to perturbations to the RCS and/or systems that are necessary to maintain the RCS in a stable and controlled condition while the RCS is in a reduced inventory condition. More specifically, this safety evaluation addresses preventing a spurious isolation of the RHR suction isolation valves MOV-*-750 and 751. This is to be accomplished by defeating the RCS high pressure interlock function of these valves when in reduced inventory condition. This interlock can lead to loss of RHR event and consequently to further reduction in RCS inventory.

Safety Evaluation Summary:

This safety evaluation shows that defeating the RHR interlocks associated with MOV-*-750 and MOV-*-751 while in modes 5 and 6 during reduced inventory operation prevents a spurious isolation of the RHR suction isolation valves MOV-*-750 and 751 and does not constitute an unreviewed safety question.

Issued: April 1, 1992

SAFETY EVALUATION: JPN-PTN-SEES-90-142 Revision 2

TEMPORARY INSTALLATION AND OPERATION OF SPARE NIS SOURCE RANGE DETECTORS

This safety evaluation addresses the Temporary System Alteration (TSA) that installs two spare source range detectors within the active region of the core which will be connected to the existing preamplifiers only in case of failure of either primary source range detector. This temporary installation and operation will supply spare monitoring capabilities. The installation and operation of the spare detectors shall only be used during fuel movement operations (Mode 6) where the safety actuation function of the source range has been defeated.

Safety Evaluation Summary:

As evaluated in this safety evaluation the implementation of the referenced TSA with the subsequent connection of the spare detector in case of single failure, does not change plant operation or safety, and does not create an unreviewed safety question or require a change to any Technical Specification. Therefore, prior NRC approval is not required.

Issued: September 4, 1991



SAFETY EVALUATION: JPN-PTN-SENS-90-145 Revision 2

TEMPORARY LIQUID RADWASTE DISCHARGE HEADER MODIFICATION

The scheduled work for Condenser and Intake Bay maintenance on Units 3 and 4 during the dual unit outage may result in a loss of all circulating water pumps for indeterminate periods of time. The liquid radwaste discharge header is interlocked with circulating water flow such that solenoid valves SV-*-1413 and SV-*-1414 will shut on loss of circulating water flow, terminating any liquid discharges. A Temporary System Alteration (TSA 3-91-61-35) will be installed to bypass the solenoid valves and discharge liquids into the ICW pump discharge will for any time all Unit 3 and 4 circulating water pumps are inoperable.

This safety evaluation addresses the effects of bypassing the solenoid valves, and using ICW pump flow for effluent dilution instead of Unit 3 and 4 circulating water (CW) pump flow for calculating the pre- and post-release isotopic Maximum Permissible Concentrations (MPC).

Safety Evaluation Summary:

This safety evaluation concluded that the temporary change did not require any changes to the Plant Technical Specifications, did not affect plant safety or operation, and did not constitute an unreviewed safety question pursuant to 10 CFR 50.59. Therefore, prior NRC approval was not required for implementation.

Issued: August 23, 1991

SAFETY EVALUATION: JNO-91-002 Revision 0

CHANGE IN CCW FLOURIDE AND CHLORIDE SPECIFICATION

This safety evaluation provides the justification for combining the current Component Cooling Water (CCW) specification for flouride (.150 ppm) and chloride (1.0 ppm) into a total flouride plus chloride specification with a maximum value of 1.0 ppm. This new specification is more conservative than the current specifications which allow a maximum value for flouride plus chloride of 1.15 ppm.

Safety Evaluation Summary:

This safety evaluation has shown that the proposed change does not result in any plant modifications or changes to the design basis of the plant. This change does not involve any unreviewed safety questions or a change to the plant Technical Specifications, pursuant to 10 CFR 50.59. In addition, this change do not adversely affect plant operation or safety. Therefore prior NRC approval was not required to implement these changes.

Issued: February 19, 1992

SAFETY EVALUATION: JPN-PTN-SENP-91-006 Revision 3

SAFETY EVALUATION FOR SAFETY FUNCTIONS IN THE MISCELLANEOUS RELAY RACKS

This safety evaluation makes an assessment of a potential 10 CFR Part 21 issue raised by a vendor with regard to safety related relays being installed in non-safety related Miscellaneous Relay Racks (MRR's). The vendor reported the potential for a substantial safety hazard to the U.S. Nuclear Regulatory Commission in letter NS-NRC-91-3603, dated June 24, 1991.

This evaluation addresses the seismic qualification of equipment racks which house safety related electrical components. This evaluation also evaluates the design of the electrical circuits contained in the MRR's which are associated with safety related functions.

Safety Evaluation Summary:

This safety evaluation identifies the safety classification for all the MRR relay's functions and provides a review of the associated electrical circuit's design. The design of those circuits has been found to meet Turkey Point requirements in regards to power supply, isolation and electrical separation. This evaluation concludes that the MRRs, installed devices and the associated circuits are within the design basis for Turkey Point. Therefore, an unreviewed safety question is not present, no change to Technical Specifications is required, and plant safety or operation is not adversely impacted.

Issued: June 16, 1992

SAFETY EVALUATION: JPN-PTN-SEEJ-91-013 Revision 1

**ENGINEERING EVALUATION FOR PERFORMANCE OF THE LOAD PROFILE TESTING
OF THE SAFETY RELATED STATION BATTERIES**

The Emergency Power System (EPS) enhancement project has performed physical modifications to the DC system which affect the load profile testing of the safety related station batteries (3D03, 3D24, 4D03, 4D24 and D52). This evaluation provides the criteria for performance of the load profile testing after installation of the EPS project.

This evaluation provides the test discharge rate, test duration and acceptance criteria for the battery service and performance tests for all of the safety related station batteries including the spare battery (3D03, 3D24, 4D03, 4D24 and D52). This evaluation also includes criteria for battery sizing.

This evaluation modifies the battery testing and acceptance criteria to be in accordance with the latest industry criteria and incorporates modifications performed with the EPS design.

Revision 1 of this evaluation addresses battery performance testing, battery sizing criteria, operator actions concerning DC load stripping, changes to service testing discharge rates and miscellaneous revisions due to the completion of the EPS enhancement project.

Safety Evaluation Summary:

This safety evaluation concludes that the above changes do not adversely affect plant operations, adversely affect plant safety, impact any Technical Specifications or involve an unreviewed safety question. Therefore, prior NRC approval was not required.

Issued: January 16, 1992

SAFETY EVALUATION: JPE-PTN-SEMS-91-015 Revision 3

RAW WATER BOOSTER PUMP REPLACEMENT SAFETY EVALUATION

The purpose of this safety evaluation is to evaluate the effects of of a Turkey Point Units 1 and 2 fossil plant initiated modification that affects a system shared by both nuclear and fossil sites and also affects the nuclear fire protection system. The modification included the replacement of the Raw Water Booster (RWB) Pumps with two higher capacity pumps with higher minimum recirculation requirements. This resulted in the addition of two individual minimum recirculation lines and associated piping and valves in lieu of the previous small common recirculation line. This evaluation addresses the applicable construction code (ANSI B31.1) and fire protection (10 CFR Appendix R) requirements and serves as a basis to revise all applicable nuclear documents.

Safety Evaluation Summary:

Revision 1 of this safety evaluation analyzed the condition of the affected systems with respect to operability and updated the nuclear plant required actions to reflect the actual fossil construction evolution that occurred.

Revision 2 adds discussions pertaining to construction code and fire protection requirements. This revision discusses major aspects of this modification that would normally be addressed in a Nuclear Engineering Package to ensure the fossil modification is also acceptable from a design perspective. This revision also addresses a recent fossil design package revision.

Revision 3 describes and justifies the final field condition following completion of all fossil construction activities. This revision also provides a status of the required actions and re-identifies the RWB pumps to be in accordance with Turkey Point Nuclear plant convention. The "temporary" RWB pump is evaluated for permanent use.

This safety evaluation concluded that the above modification is acceptable, the Technical Specifications and FSAR are not affected, and does not constitute an unreviewed safety question. Therefore, pursuant to 10 CFR 50.59, prior NRC approval was not required for implementation of this request.

Issued: April 24, 1992

SAFETY EVALUATION: JPN-PTN-SENJ-91-016 Revision 1

ENGINEERING EVALUATION RELATED TO THE PLANT GASEOUS EFFLUENT SPING MONITORS AT TURKEY POINT

During an audit conducted by the Turkey Point Quality Assurance (QA) department, it was identified that the plant vent System-Level Particulate, Iodine, and Noble Gas (SPING) air monitors did not appear to be in full compliance with the requirements of Regulatory Guide 1.97. This safety evaluation addresses the concerns identified during the audit, reestablished the design bases for these monitors and corrects inconsistencies in the plant documents.

Safety Evaluation Summary:

This safety evaluation concluded that the present design of the plant vent SPING monitoring system is in conformance with the original licensing requirements for this equipment. Since neither an unreviewed safety question nor a change to the Technical Specifications is involved, the changes to the FSAR may be made without prior NRC approval.

Issued: August 2, 1991



SAFETY EVALUATION: JPE-PTN-SEMS-91-019 Revision 2

RESIDUAL HEAT REMOVAL SYSTEM IN-SERVICE TESTING SAFETY EVALUATION.

During a recent review of the in-service testing procedures for the Component Cooling Water (CCW) pumps and the Intake Cooling Water (ICW) pumps, it was discovered that the plant should have been in an Limiting Condition for Operation (LCO) while in the test alignment. This safety evaluation examines the current Residual Heat Removal (RHR) pump in-service test procedure and provides two options to enhance the current RHR pump in service test procedure.

Safety Evaluation Summary:

This evaluation provides procedure enhancements to the current in-service test method and also qualitatively analyzes the past test configuration with respect to safety significance. This evaluation concludes that there was no significant safety issue during past pump testing. The recommended alignment does not raise any unreviewed safety questions or affect the Technical Specifications or the FSAR. Therefore, prior NRC approval is not required.

Issued: October 4, 1991



SAFETY EVALUATION: JPE-PTN-SENJ-91-035 Revision 3

EPS ENHANCEMENT PROJECT DIESEL LOADING EVALUATION

Florida Power and Light (FPL) had a vendor review/evaluate the proposed Diesel Loading Scheme for any potential impact on the current design basis accident analyses for Turkey Point Units 3 and 4. The proposed changes are a part of FP&L's Emergency Power System (EPS) Enhancement Project and result in enhancements to the way in which the electrical system handles the initial starting currents and associated voltage transients.

The scope of this safety evaluation is limited to assessing the proposed diesel loading scheme and sequencing for the FSAR Chapter 14 accident analyses and any impact on offsite thyroid doses as a result of the proposed delay in starting the Emergency Containment Filters.

Revision 2 of this safety evaluation addresses an additional 1.0 second delay to account for termination of auto/manual testing of the sequencers.

Revision 3 of this safety evaluation addresses the no LOOP case for the EPS Enhancement Project.

Safety Evaluation Summary:

This evaluation shows that the proposed diesel sequencer and loading scheme plus an additional 1.0 second delay to allow for breaker closure and measured tolerances has been determined to be acceptable with respect to the accident analyses of record for Turkey Point Units 3 and 4. Additionally, the proposed diesel sequencer and loading scheme does not represent an unreviewed safety question based upon the requirements and definitions delineated in 10 CFR 50.59 (a) (2).

Issued: December 12, 1991

SAFETY EVALUATION: JPE-PTN-SEFJ-91-039 Revision 0

SAFETY EVALUATION TO ALLOW OPERATION AT NOMINAL 50% POWER WITH EAGLE
21 G-FACTOR LIMITATION FOR TURKEY POINT UNIT 3

This safety evaluation evaluates continued operation at or below a nominal 50% power while the G-factor for delta-I penalty input to the Overtemperature delta-T Trip is changed.

Safety Evaluation Summary:

It is concluded that continued operation until calibration of the Overtemperature delta-T Trip setpoint is completed is acceptable because the review of design bases event concluded that the Power Range Trip currently set at 80% power would be actuated earlier than the Overtemperature delta-T Trip providing required protection. It was determined that an unreviewed safety question does not exist and the plant is operating within its Technical Specifications.

Issued: October 8, 1991

SAFETY EVALUATION: JPN-PTN-SEFJ-91-040 Revision 1

SAFETY EVALUATION FOR OPERATING TURKEY POINT UNITS 3 AND 4 WITH DESIGN VALUES FOR DELTA-T_o (56.1 °F) INSTEAD OF THE MEASURED VALUES IN THE OT DELTA-T AND OP DELTA-T REACTOR PROTECTION SETPOINTS

This safety evaluation was performed to demonstrate that past operation of Turkey Point Units 3 and 4 with design values of Delta-T_o (56.1°F) instead of the measured values (53.8°F) in Overtemperature Delta-T and Overpower Delta-T setpoints was still within the design basis limits for the DNBR (Departure from Nucleate Boiling Ratio) and the linear power density (kw/ft).

Safety Evaluation Summary:

Based on this safety evaluation it can be concluded that prior plant operation with a larger Delta-T_o than the actual indicated Delta-T at full power did not violate the design basis limits for DNBR and linear power density. Therefore, operation of the Units with the design value Delta-T_o did not affect plant safety or operation and does not constitute an unreviewed safety question.

Issued: May 8, 1990

SAFETY EVALUATION: JPN-PTN-SEMS-91-042 Revision 0

TURKEY POINT 3 SAFETY EVALUATION FOR PLAN OPERATION OUTSIDE TECHNICAL SPECIFICATION 3.2.1 AFD LIMITS

While performing an ex-core neutron detector calibration at 100% rated thermal power (RTP), the axial flux difference (AFD) went outside the allowable value of -6.4% for approximately two hours. This safety evaluation provides an assessment of the available margin in the measured F_0 and design DNBR during this plant evolution.

Safety Evaluation Summary:

This safety evaluation concluded that the maximum F_0 stayed below the 2.32 Technical Specification value and the minimum DNBR stayed above the limit value. Therefore, the integrity of the nuclear fuel was maintained and there was no risk to the health and safety of the public.

Issued: November 15, 1991

SAFETY EVALUATION: JPN-PTN-SENS-91-062 Revision 0

**TEMPORARY SYSTEM ALTERATION - ADDITION OF JUMPER ACROSS RESISTOR LA1
GROUND DETECTION SCHEME**

The existing 480 Volt Load Centers at Turkey Point are provided with ground detection devices. Through this device, ground detection is provided by a ground detection voltmeter located on each Load Center panel. These voltmeters will read zero volts when no ground is present. If a ground occurs, the ground detection voltmeter will have a voltage greater than zero. A selector switch can be rotated to identify which phase is grounded. Additionally, each Load Center is supplied with an annunciation circuit consisting of a series of 4 X 75 OHM, 190 watt resistors and a relay. If a ground is detected an annunciator window in the control room will light. Currently, one of the four resistors has failed open.

This safety evaluation addresses a Temporary System Alteration request (TSA 04-91-006-033) to add a jumper in place of the open resistor. The recommended change will enable the ground detection system to be operable even though one of four resistors is open.

Safety Evaluation Summary:

This safety evaluation concluded that the temporary change did not require any changes to the Plant Technical Specifications, did not affect plant safety or operation, and did not constitute an unreviewed safety question pursuant to 10 CFR 50.59. Therefore, prior NRC approval was not required for implementation.

Issued: July 30, 1991

SAFETY EVALUATION: JPN-PTN-SEMS-91-070 Revision 0

REACTOR VESSEL STUD HOLE THREAD DEVIATION

This "wrap-around" safety evaluation provides additional technical information to supplement the vendor 10 CFR 50.59 safety evaluation (SECL-91-366). The vendor evaluation determined that the maximum number of threads that could be removed (if required) from the reactor vessel stud holes without a loss of reactor vessel integrity is 15 full threads. This evaluation is based on the current condition of stud holes 35, 37, 50, & 51 but can be applied to any stud hole.

Safety Evaluation Summary:

This safety evaluation concluded that removal of a maximum of 15 threads from the reactor vessel stud holes does not impact plant safety or operation, does not result in an unreviewed safety question or require changes to the Technical Specifications. Therefore, prior NRC approval was not required pursuant to 10 CFR 50.59.

Issued: September 9, 1991

SAFETY EVALUATION: JPN-PTN-SEMS-91-071 Revision 3

SAFETY EVALUATION FOR SIS AND RCS TEMPORARY SYSTEM ALTERATIONS DURING REACTOR COOLANT SYSTEM LEAK TESTING FOR TURKEY POINT UNIT 3

As part of the Unit 3 ten year ISI hydrostatic testing activities scheduled for completion prior to the end of the Dual Unit Outage, several off-normal valving configurations will be required in the Safety Injection (SI) and Reactor Coolant (RC) Systems during Mode 3 plant conditions.

Safety Evaluation Summary:

Based on this safety evaluation, this temporary change does not constitute an unreviewed safety question or a change to any Technical Specification. Therefore, prior NRC approval was not required pursuant to 10 CFR 50.59.

Issued: September 16, 1991

SAFETY EVALUATION: JPN-PTN-SEMS-91-078 Revision 0

SAFETY EVALUATION FOR SIS AND RCS TEMPORARY SYSTEM ALTERATIONS DURING
REACTOR COOLANT SYSTEM LEAK TESTING FOR TURKEY POINT UNIT 4

As part of the Unit 4 ten year ISI hydrostatic testing activities scheduled for completion prior to the end of the Dual Unit Outage, several off-normal valving configurations will be required in the Safety Injection (SI) and Reactor Coolant (RC) Systems during Mode 3 plant conditions.

Safety Evaluation Summary:

Based on this safety evaluation, this temporary change does not constitute an unreviewed safety question or a change to any Technical Specification. Therefore, prior NRC approval was not required pursuant to 10 CFR 50.59.

Issued: October 15, 1991

SAFETY EVALUATION: JPN-PTN-SEMS-91-082 Revision 0

TURKEY POINT UNIT 4 DROPPED SPLIT WASHER IN BOTTOM MOUNTED INSTRUMENT (BMI) CONDUIT TUBE

During a recent disassembly activity of a Unit 4 seal table low pressure compression fitting (location F-6), a 3/4" diameter 1/16" thick split washer was inadvertently dropped in the annular space between the ID of the Bottom Mounted Instrument (BMI) tube and the OD of the inserted thimble tube. The worst case present location of the split washer is in the bottom horizontal span of the F-6 tube in the Unit 4 sump area at elevation -15' 8". This is approximately 13 feet below the reactor vessel bottom. The split washer was not visible after being dropped and was not retrieved. This safety evaluation addresses the effects of leaving the split washer in the F-6 BMI tube while maintaining the operability of this location.

Safety Evaluation Summary:

This safety evaluation determined that the split washer will remain in the annular space between the BMI and thimble tubes near the horizontal BMI conduit runs and can not be lifted into the reactor vessel. There are no new failure modes introduced by leaving the split washer in the BMI tube (i.e. fretting, corrosion, system operability, etc.), therefore this condition is acceptable and there are no plant restrictions or effects on plant safety.

This safety evaluation has determined that this condition does not represent an unreviewed safety question and that the Technical Specifications do not require a change. Therefore this activity does not require prior NRC approval pursuant to 10 CFR 50.59.

Issued: October 15, 1991

SAFETY EVALUATION: JPN-PTN-SECS-91-088 Revision 0

SAFETY EVALUATION ALLOWING A MAN-BASKET TO REMAIN WITHIN UNIT 4 CONTAINMENT

This evaluation addresses the acceptability of leaving a man-basket within the Unit 4 Containment Structure during all modes of operation.

The man-basket in combination with the Polar Crane was utilized during the Dual Unit Outage for maintenance activities and for valve manipulations in preparation for the integrated leak rate test (ILRT). In order to remove the man-basket, which measures 30" X 60" X 88", and is constructed of welded structural steel members, the Containment equipment hatch would be required to be opened. Due to scheduler considerations Nuclear Engineering investigated the acceptability of allowing the basket to remain within containment during all modes of operation.

The evaluation also concludes that this activity will have no adverse impact on plant operations, and will not compromise the safety and licensing bases for Turkey Point Unit 4. As such, an unreviewed safety question does not exist and a change to the Technical Specifications is not required.

Safety Evaluation Summary:

This safety evaluation concludes that the man-basket can remain within the Containment Structure during all modes of operation provided that all of the requirements stipulated within this evaluation are followed. Based on this evaluation, it is concluded that this change does not constitute an unreviewed safety question or a change to the plant Technical Specification pursuant to 10 CFR 50.59. This change has no effect on plant operation and no safety concerns. Thus prior NRC approval was not required for implementation.

Issued: October 21, 1991

SAFETY EVALUATION: JPN-PTN-SEIS-91-090 Revision 0

TEMPORARY SYSTEM ALTERATION - CONNECTION OF TEST EQUIPMENT TO AN
OPERABLE EAGLE-21 CHANNEL

This evaluation addresses the temporary connection of recording instrumentation to the Eagle 21 test panel while the protection channel is in service (in or out of the tripped mode) as requested by TSA 003-91-049-042. This safety evaluation does not provide for monitoring of multiple redundant channels simultaneously.

Safety Evaluation Summary:

This safety evaluation concluded that the temporary change did not require any changes to the Plant Technical Specifications, did not affect plant safety or operation, and did not constitute an unreviewed safety question pursuant to 10 CFR 50.59. Therefore, prior NRC approval was not required for implementation.

Issued: November 19, 1991



SAFETY EVALUATION FOR OVERTEMPERATURE (OT) AND OVERPOWER (OP) DELTA-T FOR TURBINE RUNBACK SETPOINT

This safety evaluation addresses setting the Overtemperature Delta-T (OT Delta-T) and the Overpower Delta-T (OP Delta-T) turbine runback setpoints to a value below the recommended Delta-T setting of 2% power below the trip setpoints. Setting the OT and OP turbine runback to a value below the typical recommended setting will provide assurance that the alarms from the OT Delta-T Rod Stop and the Auto turbine runback will be eliminated (reduced).

The OT Delta-T and OP Delta-T turbine runback is intended to improve plant availability by assisting the operators in mitigating a transient to prevent a reactor trip and no credit is taken in any safety analysis. The setpoint change impacts the operation of the turbine runback feature only when the rod control system is in manual for a limited number of slow developing transients. Although the time window for effective operator action has been shortened, sufficient time to mitigate a transient still exists to avoid a reactor trip.

Safety Evaluation Summary:

This safety evaluation concludes that the OT Delta-T and OP Delta-T turbine runback setpoint at Turkey Point Units 3 and 4 can be reduced to 2% of nominal loop Delta-T below the trip setpoint to reduce the number of turbine runback alarms. In the event that a turbine runback should occur while the rod control system is in manual, the operator would insert control rods to reduce reactor power to prevent a reactor trip.

The safety evaluation concludes that above change does not require a change to the Technical Specifications, does not impact plant safety or safe operation, and does not constitute an unreviewed safety question as defined by 10CFR50.59 (a) (2). Therefore, this activity did not require prior NRC review and approval.

Issued: November 22, 1991



SAFETY EVALUATION JPN-PTN-SENS-91-099 Revision 1

SAFETY EVALUATION FOR TEMPORARY SYSTEM ALTERATION - PLACING SEQUENCER TEST MODE TO OFF

The Sequencers are designed to perform a continuous automatic test and to abort the Auto-Test in response to a Valid Input. The Sequencer Auto-Test program generates minimum duration pulse signals that are used in the Programmable Logic Controller (PLC) for the purpose of verifying the PLC program and the circuit continuity. Certain failures of the PLC output test modules can result in the generation of a false "Valid Signal" that is processed by the sequencer as a field Valid Input.

Because this is potentially an undesirable situation, this safety evaluation justifies placing the test mode selector switch to "OFF" and discontinuing use of the Auto-Test. A Manual Test will be performed on a 30 day interval. Confirmation or modification of the test requirements necessary to fulfill the 30 day surveillance interval will be provided under separate documentation. The Sequencer Not In Auto Test alarm will also be defeated during this time period in accordance with TSA 4-91-24-40. These conservative measures will assure that the potential failure mode is eliminated while testing is being performed on a failed test module.

Revision 1 of this safety evaluation provides the requirements necessary to meet the 30 day surveillance interval.

Safety Evaluation Summary:

This safety evaluation concludes that disabling the Sequencer Auto Self-Test feature by placing the test selector switch in OFF does not present an unreviewed safety question or require a change to the Technical Specifications, and does not impact plant operation or safety. Therefore, prior NRC approval was not required pursuant to 10 CFR 50.59.

Issued: January 23, 1992

SAFETY EVALUATION JPN-PTN-SEFJ-92-001 Revision 0

SAFETY EVALUATION FOR EDDY CURRENT TESTING OF NUCLEAR FUEL AT TURKEY POINT UNIT 4

This safety evaluation reviews the spent fuel rod inspection at Turkey Point Unit 4, using a Spent Fuel Rod Inspection Stand. The potential safety issues associated with performing this inspection are enveloped by the fuel handling analyses previously evaluated in the FSAR.

Safety Evaluation Summary:

This evaluation shows that the eddy current testing of the nuclear fuel at Turkey Point Units 4 does not constitute an unreviewed safety question, does not require a change to the Technical Specifications, and does not impact plant operation or safety. Therefore, prior NRC approval was not required pursuant to 10 CFR 50.59.

Issued: January 10, 1992

SAFETY EVALUATION JPN-PTN-SECP-92-004 Revision 1

UNIT 3 TWENTIETH YEAR CONTAINMENT TENDON SURVEILLANCE

The Technical Specifications, Section 4.6.1.6.1 requires that containment tendon surveillance be performed at one, three and five years (and every five years thereafter) after the initial containment structural integrity test. The containment structure tendon surveillance program is a systematic means of assessing the continued performance of the containment post-tensioning system.

The surveillance and construction activities associated with the Unit 3 twentieth year tendon surveillance are restricted such that design basis structural integrity is maintained. Therefore, surveillance activities may be performed during power operation.

This safety evaluation is provided to address construction and surveillance activities associated with the twentieth year tendon surveillance. The scope of the safety evaluation includes safety assessments for the following:

1. mobilization activities
2. platform erection
3. use of stressing ram and platforms
4. handling and rigging heavy loads
5. extraordinary weather precautions
6. de-mobilization activities

Revision 1 to this safety evaluation provides for inspection activities which satisfy Technical Specification requirements for the twentieth year surveillance, and also allows for additional inspection activities to be performed, but which are not required to satisfy Technical Specification requirements.

Safety Evaluation Summary:

The evaluation concluded that this activity does not constitute an unreviewed safety question and does not require changes to the Technical Specification to accomplish this tendon surveillance. Therefore, prior NRC approval for implementation of this activity is not required. Furthermore, the supporting analyses and governing controls assure that performing the tendon surveillance has no adverse effect on plant safety or safe operation

Issued: June 10, 1992

SAFETY EVALUATION JPN-PTN-SEEP-92-005 Revision 0

SAFETY EVALUATION FOR OPERATION OF MOTOR DRIVEN INSTRUMENT AIR COMPRESSOR

Turkey Point has recently determined that it is desirable to operate the installed electric motor driven Instrument Air Compressors (IACs) C1, 3C1 and 4C1 which have been deenergized via administrative control since 1986. Accordingly, this safety evaluation evaluates the acceptability of operating the installed motor driven IACs C1 and 3C1 in conjunction with their associated diesel driven air compressors. This evaluation, also, confirms the acceptability of operation of IAC 4S and evaluates the acceptability of operating the installed motor driven IAC 4C1 in conjunction with the IAC 4S and its associated diesel driven air compressors.

Safety Evaluation Summary:

Operation of the motor driven IACs and/or diesel driven air compressors in accordance with this safety evaluation does not have an adverse effect on plant safety, security or operation, does not constitute an unreviewed safety question, and does not require changes to the Technical Specifications. Therefore, prior NRC approval for implementation is not required.

Issued: March 13, 1992



SAFETY EVALUATION JPN-PTN-SENS-92-006 Revision 0

SAFETY EVALUATION FOR LOWERING THE SETPOINT OF THE RCP MOTOR LOWER GUIDE BEARING TEMPERATURE ALARM

This safety evaluation addresses the change which lowers the setpoint associated with the Reactor Coolant Pump (RCP) motor lower guide bearing temperature alarm closer to the normal operating temperature of the bearings. Lowering the alarm setpoint will give the operators more time to respond to a potential overtemperature condition for the RCPs.

Safety Evaluation Summary:

This evaluation concludes that the setpoint change has no effect on plant safety or operation. The change was evaluated under the provisions of 10 CFR 50.59 and concludes that there is no unreviewed safety question nor are changes required to the Technical Specifications.

Issued: January 28, 1992

SAFETY EVALUATION: JPN-PTN-SENP-92-009 Revision 0

SUBSTANTIAL SAFETY HAZARDS EVALUATION RELATED TO PRESSURIZER VENTS AT COLD SHUTDOWN

Vendor letter FPL-92-529 notifying Florida Power and Light of a potential 10 CFR 21 concern involving the use of pressurizer vents during cold shutdown. This evaluation reviews the issues associated with the Westinghouse letter and the adequacy of level instrumentation associated with reduced inventory operation with respect to INPO SOER 88-3, recommendation 4.

The above letter identified a potential issue which relates to limitations of pressurizer venting for decay heat removal at cold shutdown. During a loss of RHR cooling event, there is a potential for water hold-up in the pressurizer if the reactor coolant is allowed to boil. It is possible that the steam velocity in the surge line would be high enough that liquid entrainment prevents water from draining back from the pressurizer into the hot leg. This effect is also called surge line flooding.

This Substantial Safety Hazards Evaluation serves two functions. First, it evaluates the Westinghouse letter with respect to the plant specific conditions at Turkey Point. The Substantial Safety Hazards Evaluation portion concludes that a substantial safety hazard does not exist and is not reportable under 10 CFR 21. Second, the 10 CFR 50.59 safety evaluation portion allows for certain procedure changes that govern reduced inventory operation to be made to enhance the operator's ability to cope with a loss of RHR event.

Safety Evaluation Summary:

The procedure changes recommended in this evaluation provide cautions and notes that provide more information to the operations personnel during loss of RHR events. The procedure changes do not alter the manner or the methodology employed to mitigate loss of RHR events. The safety evaluation concludes that the proposed procedure changes do not require a change to any Technical Specification, did not impact the safety and operation of the plant, and does not constitute an unreviewed safety question. Therefore, prior NRC approval is not necessary.

Issued: May 12, 1992

SAFETY EVALUATION JPN-PTN-SEMS-92-012 Revision 0

SAFETY EVALUATION FOR TEMPORARY SYSTEM ALTERATION (TSA) REPLACING CHARGING PUMP 4B RELIEF VALVE RV-4-283B WITH A BLIND FLANGE

This Safety Evaluation addresses the temporary replacement of Charging Pump Relief Valve RV-4-283B with blind flanges in accordance with TSA 04-92-47-3 to eliminate a potential leakage path through the valve during troubleshooting. Testing performed previously has identified leakage of about 5 gpm through one or more of three flow paths associated with the 4B Charging Pump. Eliminating the relief valve as a leakage path allows for checking of the other two potential flow paths.

The pressure integrity of the Charging System is maintained by the TSA and an operator will remain in the area while the 4B charging pump (and blind flanges) are exposed to system pressure. Removal of the relief valve is acceptable from an overpressure protection standpoint because the 4B pump will not be operated.

During the last In Service Test on the 4A charging pump the flow was inadequate with the 4B pump valved in; however, with the 4B pump isolated, the 4A pump's flow was found to be acceptable. To be conservative, the 4A charging pump will be declared out of service while the 4B pump is valved in. Considering the 4C charging pump to be the only operable charging pump causes Unit 4 to be in a 72 hour LCO.

Safety Evaluation Summary:

This safety evaluation concludes that the temporary system alteration does not require any changes to the Plant Technical Specifications, does not affect plant safety or operation, and does not involve an unreviewed safety question pursuant to 10 CFR 50.59. Therefore, prior NRC approval was not required prior to implementation.

Issued: February 24, 1992

SAFETY EVALUATION: JPN-PTN-SEIS-92-013 Revision 0

**SAFETY EVALUATION FOR OVERTEMPERATURE AND OVERPOWER DELTA-T
TURBINE RUNBACK SETPOINT CHANGE**

On Friday 11/15/91, Florida Power & Light - Turkey Point Unit 3, the Delta-T. parameter used in the Overtemperature/Overpower (OT/OP) Delta-T (DT) setpoint calculation was normalized to 100% power at actual 100% power plant conditions. During this time the plant began to experience alarms in the OT Delta-T Rod Stop and the Auto Turbine Runback Rod Block on Unit 3 loop B circuit with no significant consequence to plant operation. Turbine runback requires 2-out-of-3 loops for actuation. The alarm was caused by fluctuations of the Delta-T signal (Delta-T lead/lag minus the OTDT trip setpoints) which were occasionally reaching the turbine runback setpoint. PC/M 91-191 and 91-192 were issued to change the turbine runback setpoints to 1.1°F below their respective trip setpoints. However, several spurious alarms have been received after implementation of the above PC/Ms. This safety evaluation addresses plant operation with the OTDT and OPDT turbine runback setpoints set equal to their respective reactor trip setpoints.

Safety Evaluation Summary:

The turbine runback on OTDT or OPDT is for plant availability only. As such, the turbine runbacks are considered quality related. Credit is not taken for turbine runback on a OTDT or OPDT signal in any FSAR accident analyses.

The results of this safety evaluation show the requested change to the OTDT and OPDT turbine runback setpoints does not have an adverse effect on plant operation or safety, does not constitute an unreviewed safety question, and does not require changes to the Technical Specifications. Therefore, prior NRC approval was not required pursuant to 10 CFR 50.59.

Issued: April 2, 1992



SAFETY EVALUATION: JPN-PTN-SEIS-92-014 Revision 0

SAFETY EVALUATION FOR HIGH DELTA-T ALARM SETPOINT CHANGE

While preparing scaling drawings for the Eagle-21 System, it was discovered that the High Delta-T alarm setpoint may not be set to the proper value. The High Delta-T alarm is controlled by comparators TC-412A2, TC-422A2 and TC432A2 for Loops A, B and C respectively. These comparators are calibrated using Plant Procedures 3/4-PMI-041.6, 7, 8. Currently the acceptance criteria given by these procedures require that the setpoint for these comparators be set to 58.8°F. The current setpoint is derived from multiplying 1.05 times the original plant design Delta-T_o of 56.1°F, ($1.05 \times 56.1 = 58.8$). However, during a calibration of the Eagle-21 System at 50% reactor power in Turkey Point Unit 3, it was discovered that the design Delta-T_o was being used instead of the required measured Delta-T_o during calorimetrics. Safety evaluation JPN-PTN-SEFJ-91-040 evaluated the effect of using the original plant design Delta-T_o on the reactor trip logic. This safety evaluation addresses the current High Delta-T alarm setpoint and provides for revision of the High Delta-T alarm setpoint using the required measured Delta-T_o.

Safety Evaluation Summary:

The High Delta-T alarm is intended to provide the operator with a warning of an impending Overtemperature or Overpressure Delta-T reactor trip. As such, the High Delta-T alarm is considered Quality Related. Credit is not taken for High Delta-T alarm signal in FSAR accident analysis and the setpoint change introduces no adverse effect on plant operation or safety. The results of this safety evaluation show that operation with the revised High Delta-T alarm setpoint does not constitute an unreviewed safety question and does not require any Technical Specification changes. Therefore, prior NRC approval is not required pursuant to 10 CFR 50.59.

Issued: April 2, 1992



SAFETY EVALUATION: JPN-PTN-SEIS-92-027 Revision 0

SAFETY EVALUATION FOR CONNECTION OF TEST EQUIPMENT ON SPARE RELAY CONTACTS

This safety evaluation is for the temporary connection of unattended recording instrumentation to be used for troubleshooting and data monitoring. The temporary recording instrumentation will be connected to various relays' spare contacts in Control Room "Control and Protection Relay Racks" while the relay is in service. This safety evaluation does not provide for monitoring redundant channels simultaneously. This evaluation is applicable to both Units 3 and 4.

Safety Evaluation Summary:

This safety evaluation concluded that this change does not require any changes to the Plant Technical Specifications, does not affect plant safety or operation, and does not constitute an unreviewed safety question pursuant to 10 CFR 50.59. Therefore, prior NRC approval was not required for implementation of this change.

Issued: May 5, 1992

SAFETY EVALUATION: JPN-PTN-SENS-92-028 Revision 1

TURKEY POINT UNIT 3 FREEZE SEAL SAFETY EVALUATION FOR REPAIR OF VALVE CV-3-310A

This safety evaluation addresses the use of freeze seals in order to repair valve CV-3-310A. This valve is a 3" globe valve in the primary charging flow path downstream of the regenerative heat exchanger. The maintenance will be performed while Unit 3 is in cold shutdown (Mode 5).

Valve CV-3-310A is classified as Quality Group A on 5610-T-E-4505, Sheet 1. Therefore this safety evaluation is classified as Safety Related.

Revision 1 to this evaluation confirms that the original evaluation basis remains valid for Reactor Coolant System pressures up to 400 psig.

Safety Evaluation Summary:

Based on this evaluation and the stated requirements, this maintenance activity will have no adverse impact on plant operation or safety, does not involve an unreviewed safety question or change to the Technical Specifications pursuant to 10CFR50.59. Therefore, prior NRC approval is not required for implementation of this activity.

Issued: May 6, 1992



SAFETY EVALUATION: JPN-PTN-SEMS-92-030 Revision 1

STEAM GENERATOR MOISTURE CARRYOVER TESTING.

This safety evaluation provides the justification to support the use of Temporary Procedures TP-840 (Unit 3) and TP-841 (Unit 4) for testing the moisture carryover of the Turkey Point Units 3 & 4 steam generators. These tests will be used to determine the operating condition of the internal steam generator moisture removal equipment. The testing being performed is a standard moisture carryover test which has been performed at numerous nuclear power plants throughout the country.

Revision 1 clarifies the safety function of the pressure transmitters which will be used as sample connection points for the carryover test. Additionally, clarification for the use of AFW for the carryover test is provided.

Safety Evaluation Summary:

This safety evaluation concluded that the implementation of these temporary procedures does not impact plant safety or operation, does not constitute an unreviewed safety question, and does not require changes to the Technical Specifications. Therefore, pursuant to 10 CFR 50.59, NRC approval is not required prior to implementation.

Issued: June 5, 1992



SECTION 3
RELOAD SAFETY EVALUATIONS

RELOAD SAFETY EVALUATIONS

PLANT CHANGE/MODIFICATION 91-018

TURKEY POINT UNIT 4 CYCLE 13 FUEL RELOAD

This engineering package (EP), provides the reload core design of the Turkey Point Unit 4 Cycle 13 developed by Florida Power & Light Co. The Cycle 13 energy requirement is 10650 EFPH based on a Cycle 12 length of 9790 EFPH.

The primary design change to the core for Cycle 13 is the replacement of 64 irradiated assemblies with 48 fresh Region 15 fuel assemblies and 16 irradiated assemblies re-inserted from previous cycles. The fuel is arranged in a low leakage pattern with no significant differences between the Cycle 13 loading pattern and the Cycle 12 design. Region 15 incorporates Debris Resistant Fuel Assemblies (DRFA) which use longer fuel rod bottom endcaps and relocated grids. This design was first used at FPL in Turkey Point Unit 3 Cycle 12. In addition, there are several other minor design changes which include:

- 1) an increase in the radius of the fuel rod bottom end plug to facilitate rod loading;
- 2) addition of a skirt to the bottom nozzle for increased strength;
- 3) implementation of an anti-snag outer grid strap into the zircaloy grids; and
- 4) updated WABA assembly which reflect the latest design features for this component.

The safety analysis of this design was performed by Westinghouse and independently reviewed by Florida Power and Light Co.

Supplement 1:

Supplement 1 to the EP adds curves of RCS boron concentration required to maintain shutdown margin as a function of temperature and exposure to support operation with a reduced boric acid concentration in the boric acid tanks. These curves provide operational guidance to ensure that shutdown margin is maintained at various RCS temperatures during reactor cooldowns. This supplement does not affect, amend, or change the original safety evaluation or any Technical Specifications.

Safety Evaluation Summary

This safety evaluation concluded that implementation of this EP will not adversely impact plant safety or operation. Cycle 13 operation does not pose an unreviewed safety question and can be implemented with no changes to the Turkey Point Technical Specifications. Therefore, prior NRC approval is not required for implementation.

SECTION 4

ANNUAL REPORT OF POWER OPERATED RELIEF VALVE (PORV) ACTUATIONS



ANNUAL REPORT OF SAFETY AND RELIEF VALVE CHALLENGES

By letter dated June 18, 1980, (L-80-186) Florida Power and Light stated the intent to comply with the requirements of item II.K.3.3 of Enclosure 3 to the commissioner's letter of May 7, 1980 (Five Additional TMI-2 Related Requirements for Operating Reactors).

The following is a list of power operated relief valve (PORV) actuations for Turkey Point Units 3 and 4 from July 1, 1991 to June 30, 1992.

Procedure Title Key

3-OP-041.4 and 4-OP-041.4	Overpressure Mitigation System
3-OSP-041.4 and 4-OSP-041.4	Overpressure Mitigation System Nitrogen Backup Leak and Functional Test
OP 0209.1	Valve Exercising Procedure

Unit 3

August 12, 1991	PORVs 455C and 456 were cycled per 3-OSP-041.4
September 1, 1991	PORVs 455C and 456 were cycled per 3-OP-041.4
January 3, 1992	PORV 455C was cycled during performance of 3-OSP-041.3, Pressurizer Heater Operational Test, due to an increase in the pressurizer pressure controller demand signal
April 28, 1992	PORVs 455C and 456 were cycled per 3-OP-041.4
May 5, 1992	PORVs 455C and 456 were cycled per 3-OP-041.4

Unit 4

October 6, 1991	PORVs 455C and 456 were cycled per 4-OP-041.4
October 6, 1991	PORVs 455C and 456 were cycled per 4-OSP-041.4
October 20, 1991	PORVs 455C and 456 were cycled per 4-OP-041.4
December 11, 1991	PORV 455C was cycled per 4-OSP-041.4
December 11, 1991	PORVs 455C and 456 were cycled per 4-OP-041.4
December 14, 1991	PORVs 455C and 456 were cycled per 4-OP-041.4
January 29, 1992	PORVs 455C and 456 were cycled per 4-OSP-041.4



SECTION 5

STEAM GENERATOR TUBE INSPECTIONS

THERE WERE NO STEAM GENERATOR TUBE INSPECTIONS PERFORMED
FOR UNITS 3 OR 4 DURING THIS REPORTING PERIOD