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MORRONI,M. Public Service Electric & Gas Co. of New Jersey
MILLER,L.K. Public Service Electric & Gas Co. of New Jersey
RECIP.NAME RECIPIENT AFFILIATION

SUBJECT: Monthly operating rept for Jul 1990 for Salem 2.W/900821 ltr.

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Public Service Electric and Gas Company P.O. Box E Hancocks Bridge, New Jersey 08038

Salem Generating Station

August 15, 1990

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

Dear Sir:

MONTHLY OPERATING REPORT
SALEM NO. 2
DOCKET NO. 50-311

In compliance with Section 6.9.1.6, Reporting Requirements for the Salem Technical Specifications, the original copy of the monthly operating reports for the month of July 1990 are being sent to you.

Average Daily Unit Power Level
Operating Data Report
Unit Shutdowns and Power Reductions
Safety Related Maintenance
Major Plant Modifications
Operating Summary
Refueling Information

Sincerely yours,

V J Palugio
for L K Miller

L. K. Miller
General Manager -
Salem Operations

RH:pc

cc: Mr. Thomas T. Martin
Regional Administrator USNRC
Region I
631 Park Avenue
King of Prussia, PA 19046

Enclosures

8-1-7.R4

IE24

AVERAGE DAILY UNIT POWER LEVEL

Docket No.	50-311
Unit Name	Salem # 2
Date	08-10-90
Completed by	Art Orticelle
Telephone	609-339-2122

Month JULY, 1990

Day	Average Daily Power Level (MWe-NET)	Day	Average Daily Power Level (MWe-NET)
1	0	17	0
2	0	18	0
3	0	19	0
4	0	20	0
5	0	21	0
6	0	22	0
7	0	23	0
8	0	24	0
9	0	25	0
10	0	26	0
11	0	27	0
12	0	28	0
13	0	29	0
14	0	30	0
15	0	31	0
16	0		

[illegible]

CORRECTED OPERATING DATA REPORT

Docket No: 50-311
Date: 08-10-90
Completed by: Art Orticelle
Telephone: 609-339-2122

Operating Status

			Notes
1.	Unit Name	Salem No. 2	
2.	Reporting Period	JULY 1990	
3.	Licensed Thermal Power (MWt)	3411	
4.	Nameplate Rating (Gross MWe)	1170	
5.	Design Electrical Rating (Net MWe)	1115	
6.	Maximum Dependable Capacity(Gross MWe)	1149	
7.	Maximum Dependable Capacity (Net MWe)	1106	

8. If Changes Occur in Capacity Ratings (items 3 through 7) since Last Report, Give Reason N/A

9. Power Level to Which Restricted, if any (Net MWe) N/A

10. Reasons for Restrictions, if any N/A

	This Month	Year to Date	Cumulative
11. Hours in Reporting Period	744	5087	77136
12. No. of Hrs. Reactor was Critical	21.9	2161.0	48166.9
13. Reactor Reserve Shutdown Hrs.	0	0	0
14. Hours Generator On-Line	0	2056.2	46601.5
15. Unit Reserve Shutdown Hours	0	0	0
16. Gross Thermal Energy Generated (MWH)	1797.6	6683479.2	95735218.6
17. Gross Elec. Energy Generated (MWH)	0	2266270	48313322
18. Net Elec. Energy Generated (MWH)	-13760	2138224	45938130
19. Unit Service Factor	0	40.4	60.4
20. Unit Availability Factor	0	40.4	60.4
21. Unit Capacity Factor (using MDC Net)	0	38	53.8
22. Unit Capacity Factor (using DER Net)	0	37.7	53.4
23. Unit Forced Outage Rate	0	10.0	25.4

24. Shutdowns scheduled over next 6 months (type, date and duration of each)

None

25. If shutdown at end of Report Period, Estimated Date of Startup:

08/14/90

UNIT SHUTDOWN AND POWER REDUCTIONS
REPORT MONTH JULY 1990

DOCKET NO.: 50-311
UNIT NAME: Salem #2
DATE: 08-10-90
COMPLETED BY: Art Orticelle
TELEPHONE: 339-2122

NO.	DATE	TYPE [1]	DURATION (HOURS)	REASON [2]	METHOD OF SHUTTING DOWN REACTOR	LICENSE EVENT REPORT #	SYSTEM CODE [4]	COMPONENT CODE [5]	CAUSE AND CORRECTIVE ACTION TO PREVENT RECURRENCE
0054	07/01/90	S	744	A	4	———	CD	INSTRU	21-24 MS167 VALVES

1
F: Forced
S: Scheduled

2
Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

3
Method:
1-Manual
2-Manual Scram
3-Automatic Scram
4-Continuation of Previous Outage
5-Load Reduction
9-Other

4
Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

5
Exhibit 1 - Same Source

SAFETY RELATED MAINTENANCE
MONTH: - JULY 1990

DOCKET NO: 50-311
UNIT NAME: SALEM 2
DATE: AUGUST 10, 1990
COMPLETED BY: M. MORRONI
TELEPHONE: (609) 339-2068

WO NO	UNIT	EQUIPMENT IDENTIFICATION
880627079	2	2MISC-MECH-OUTER PENETRATION FAILURE DESCRIPTION: GROUND WATER LEAK - REPAIR
890724021	2	21WG032 FAILURE DESCRIPTION: REPLACE DIAPHRAGM
900501262	2	SPOOL RD-2-SW-740 FAILURE DESCRIPTION: HOLE IN WELD - REPAIR
900611130	2	22RCP SEAL OUT TEMP FAILURE DESCRIPTION: INDICATION READING LOW - INVESTIGATE
900618122	2	TAC MOTOR AMPS INDICATION FAILURE DESCRIPTION: INDICATION FAILED HIGH - INVESTIGATE
900722049	2	2A D/G SPEED CONTROL FAILURE DESCRIPTION: SPEED-LOAD CONTROL SLOW RESPONSE - TROUBLESHOOT
900724121	2	21 SW PUMP FAILURE DESCRIPTION: PUMP WILL NOT RESTART - TROUBLESHOOT AND REWORK

[illegible]

MAJOR PLANT MODIFICATIONS
MONTH: - JULY 1990

DOCKET NO: 50-311
UNIT NAME: SALEM 2
DATE: AUGUST 10, 1990
COMPLETED BY: M. MORRONI
TELEPHONE: (609) 339-2068

*DCR	PRINCIPAL SYSTEM	DESCRIPTION
2EC-2122	Reactor Containment Fan Cooler Motors	This design change provided for refurbishment of two Reactor Containment Fan Cooler Unit (RFCU) motors to improve their reliability.
2EC-2187	Chemical Volume & Control System	This design change reduced the boron concentration in the Boron Injection Tank (BIT) from 21,000 ppm to less than 7,000 ppm. The recirculation lines between the BIT and the Boron Addition Tank (BAT) were cut and capped and the heat tracing was disconnected.
2EC-2302	UHF Radio	This design change increased the coverage of the UHF radio communication system by adding four antennas.
2SM-0208	TAC Expansion Tank	This modification installed a permanent sightglass for the Turbine Auxiliaries Cooling Makeup and Expansion Tank.

* DCR - Design Change Request

MAJOR PLANT MODIFICATIONS
MONTH: - JULY 1990

DOCKET NO: 50-311
UNIT NAME: SALEM 2
DATE: AUGUST 10, 1990
COMPLETED BY: M. MORRONI
TELEPHONE: (609) 339-2068

*DCR	SAFETY EVALUATION 10 CFR 50.59
2EC-2122	This design change provided for refurbishment of two RFCU motors to improve their reliability. Certificate and qualification documentation, including workmanship qualification and seismic requalification to the original specifications and requirements, are provided by the vendor. There is no change to any plant process or discharge or to the environmental impact of the plant. No unreviewed safety or environmental questions are involved.
2EC-2187	This design change reduced the boron concentration in the BIT from 21,000 ppm to less than 7,000 ppm. The recirculation lines between the BIT and the Boron Addition Tank (BAT) were cut and capped and the heat tracing was disconnected. These modifications eliminated the Technical Specifications associated with the BIT as well as the maintenance problems associated with high concentration boron. These modifications did not increase the probability of any accidents previously evaluated or introduce any new accidents. There is no change to any plant process or discharge or to the environmental impact of the plant. No unreviewed safety or environmental questions are involved.
2EC-2302	This design change increased the coverage of the UHF radio communication system by adding four antennas. The probability of occurrence or the consequences of an accident or malfunction of equipment important to safety has not been increased. This modification has not created the possibility of an accident or malfunction different than any evaluated in the FSAR. The margin of safety defined in the Technical Specifications and the FSAR has not been reduced. There is no change to any plant process or discharge or to the environmental impact of the plant. No unreviewed safety or environmental questions are involved.
2SM-0208	This modification installed a permanent sightglass for the TAC Makeup and Expansion Tank. There is no change to any plant process or discharge or to the environmental impact of the plant. No unreviewed safety or environmental questions are involved.

* DCR - Design Change Request

[illegible]

MAJOR PLANT MODIFICATIONS
MONTH: - JULY 1990

DOCKET NO: 50-311
UNIT NAME: SALEM 2
DATE: AUGUST 10, 1990
COMPLETED BY: M. MORRONI
TELEPHONE: (609) 339-2068

(cont'd)

*DCR	SAFETY EVALUATION 10 CFR 50.59
2SM-0973	This design change raised the setpoint of the BIT pressure alarm from 2250 psig to 2610 psig to eliminate false alarms. The BIT high discharge pressure alarm provided the Control Room Operators a way of determining that there was a blocked flowpath during safety injection. The function of the high discharge pressure alarm has not been changed and the boric acid concentration in the BIT has been reduced to the point that boric acid solidification is no longer a concern. The new setpoint is still below the design pressure of the BIT and piping, including instrument inaccuracies. There is no change to any plant process or discharge or to the environmental impact of the plant. No unreviewed safety or environmental questions are involved.

* DCR - Design Change Request

SALEM GENERATING STATION
MONTHLY OPERATING SUMMARY - UNIT 2
JULY 1990

SALEM UNIT NO. 2

The Unit entered the period shutdown, following a reactor trip on June 28, 1990 due to failure of the 2F 460V transformer. During the transient, two Main Steam Isolation Valves (MSIVs) failed to operate as expected. An investigation into the cause identified a weakness in the MSIV isolation logic. An evaluation, performed in support of a proposed design change on the MSIVs indicated that it could not be ensured the valves would close during certain postulated main steam line pipe breaks. The Unit remained shutdown throughout the rest of the period while additional analyses and evaluations were performed.

REFUELING INFORMATION
MONTH: - JULY 1990

DOCKET NO: 50-311
UNIT NAME: SALEM 2
DATE: AUGUST 10, 1990
COMPLETED BY: M. MORRONI
TELEPHONE: (609) 339-2068

MONTH JULY 1990

1. Refueling information has changed from last month:
   YES X NO _____

2. Scheduled date for next refueling: November 30, 1991

3. Scheduled date for restart following refueling: February 9, 1992

4. a) Will Technical Specification changes or other license amendments be required?:
   YES _____ NO _____

   NOT DETERMINED TO DATE x

   b) Has the reload fuel design been reviewed by the Station Operating Review Committee?:
   YES _____ NO x

   If no, when is it scheduled?: October 1991

5. Scheduled date(s) for submitting proposed licensing action:
   N/A

6. Important licensing considerations associated with refueling:

7. Number of Fuel Assemblies:

   a. Incore 193
   b. In Spent Fuel Storage 316

8. Present licensed spent fuel storage capacity: 1170

   Future spent fuel storage capacity: 1170

9. Date of last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity: March 2003

8-1-7.R4